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UNITED STATES AIR FORCE IERA

Remedial Action and Final Radiological Status, 1964 B-58 Accident Site, Grissom Air Reserve Base, Bunker Hill, Indiana

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December 2000

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(DU). AFIERA developed a remedial action plan for the site [IERA-SD-TR-2000-0011 (Available from NTIS)]. The goal was to select areas for remediation where surface soils DU contamination levels exceeded 6.4 pCi/g of DU, with a final status goal not to exceed 8 pCi/g. This report describes remedial actions completed and final site status. An area of 300 square meters was remediated, with excavated volume estimated at 105 cubic meters. The residual DU on the site is estimated at 1.3 pCi/g, based on final status soil samples. The remedial action met all goals. Although the installation has no present interest in utilization of the site for other purposes, the site meets unrestricted public use criteria recommended by the Environmental Protection Agency. No further remedial actions are recommended for the site.

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List of Abbreviations and Acronyms

241		000/040			
²⁴¹ Am	americium-241	^{239/240} Pu	plutonium-239 & -240		
²⁴¹ Pu	plutonium-241	⁹⁰ Sr	strontium-90		
²³⁴ Th	thorium-234	^{234}U	uranium-234		
^{235}U	uranium-235	^{238}U	uranium-238		
cm	centimeter	CI	confidence interval		
CV	coefficient of variation	dpm	disintigration per minute		
DU	depleted uranium	γ	gamma		
g	gram	ft	feet		
GPS	geopositional system	HQ	Headquarters		
in	inch	kcpm	kilo counts per minute		
lbs	pounds	m	meter		
μg G ⁻¹	microgram per gram	μCi g	microcuries per gram		
mg L ⁻¹	milligram per liter	mrem y ⁻¹	millirem per year		
N	north	NA	not applicable		
pCi g ⁻¹	picocuries per gram	QA	quality assurance		
QC	quality control	RPD	relative percent difference		
s	second	σ	sigma		
Tl	thallium	V	volts		
W	west	у	year		
AFBCA	Air Force Base Conversion Agency		•		
AFIERA	Air Force Institute for Environment, Sa	fety, Occupation	nal Health Risk Analysis		
ALARA	as low as is reasonably achievable	•	,		
ARW	Air Refueling Wing				
BRAC	Base Realignment and Closure				
CFR	Code of Federal Regulations				
EPA	Environmental Protection Agency				
IDH	Indiana Department of Health				
IDEM	Indiana Department of Environmental F	Protection			
MARSSIM	Multi-Agency Radiation Survey and Sit	te Investigation	Manual		
MDC	minimal detectable concentration				
NRC	Nuclear Regulatory Commission				
ORI	Operational Readiness Inspection				
OSC	Operations Support Command				
RASS	remedial action support survey		•		
TCLP	toxicity characteristic leaching procedure				
WGP	weapons grade plutonium				

Definitions

<u>ALARA</u>: the practice of reducing radiation dose levels below specified limits using a cost-benefit analysis.

<u>Curie</u>: a unit of radioactivity equivalent to 3.7×10^{10} nuclear transformations (also referred to as disintegration) per second. One pCi is equivalent to 10^{-12} Ci or 0.037 nuclear transformations per second.

dose equivalent: for the purposes of this report, the terms dose equivalent, effective dose equivalent, and dose will be used interchangeably. The effective dose equivalent is the sum of the weighted dose equivalents for irradiated tissues or organs from ionizing radiation sources. It takes into account the different mortality risks from cancer and the risk of severe hereditary effects. A common unit for dose equivalent in the US is the mrem. For comparison to remediation criteria discussed in the report, the average American annually receives about 350 mrem from naturally occurring sources of radiation in the environment.

enriched uranium: uranium with a higher ²³⁵U mass fraction than that of natural uranium.

<u>half-life</u>: the period of time required for any given isotope to decrease to one-half of its original quantity.

<u>isotopes</u>: variation in the number of neutrons in the nuclei of atoms of the same element. For example, three common isotopes of uranium: ²³⁴U, ²³⁵U, and ²³⁸U only differ in the number of neutrons in the nucleus. Isotopes of the same element generally behave the same chemically, but can have significantly different nuclear properties (i.e., nuclear interactions and radioactivity).

minimal detectable concentration: the smallest concentration of radioactivity that can be measured under specified conditions.

pH: the negative logarithm of the effective hydrogen concentration in gram equivalents.

<u>radioactivity</u>: a property exhibited by some nuclei undergoing spontaneous nuclear transformation that has accompanying radioactive particle and/or electro-magnetic emissions.

<u>remedial action support survey</u>: a survey accomplished after a remedial action to verify the effectiveness of an action.

<u>secular equilibrium</u>: a condition where a decay product (commonly called "daughter") isotope has a very short radiological half-life compared to the radiological half-life of the "parent" isotope. In these cases, the radioactivity of the parent and daughter will be the same.

weapons grade plutonium: artificially produced type of plutonium that by mass is predominately the fissile ²³⁹Pu. Fissile isotopes of elements are capable of forming critical masses of material necessary for a nuclear chain reaction.

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1. Introduction

a. Purpose. This report describes the remedial action work and final status survey accomplished at the B-58 accident site on Grissom Air Reserve Base (ARB). Remediation actions described in this report were limited to a 300 m² site in a grassy area alongside NE-SE Runway 23 and were accomplished by Cabrera Remediation Services Inc under contract to the US Army Operations Support Command (OSC). The 300 m² site was the only contaminated area within a considerably larger survey area based on the characterization survey (Rademacher and Hoak 2000). Radiological surveys and supervision of the remedial actions were accomplished by the Radiation Surveillance Division of the AF Institute for Environment, Safety and Occupational Health Risk Analysis (AFIERA). The Indiana Departments of Health and Environmental Management (IDH and IDEM) reviewed survey and workplan documents for the site, and participated in field activities as an independent observer. The site was contaminated by a nuclear weapons accident that occurred 8 December 1964 when a B-58 strategic bomber skidded off the runway and caught fire. In 1964, the B-58 wreckage and some contaminated soils from the accident site were buried at another location that is now part of the Base Realignment and Closure (BRAC) Commission property. The aircraft burial site investigation is being accomplished by the Air Force Base Conversion Agency (AFBCA) and is described in other documents. The remedial action goals described in the remedial action plan for the accident site (Rademacher 2000), including soil removal, in-situ γ-measurements, and soil sampling were met. Although the 434th Air Refueling Wing (ARW) plans to retain the property, the site meets an unrestricted public release dose rate criterion of 15 mrem y⁻¹ recommended by the Environmental Protection Agency (EPA).

b. Site Description.

Grissom AFB was realigned under the BRAC Commission and was renamed Grissom ARB. Grissom ARB maintains military planes and the runway with the 434th ARW. The Wing is equipped with 22 KC-135 aircraft and 1,300 personnel. The accident site is alongside NE-SW Runway 23 (Figures 1 and 2). The area is bounded by a concrete runway and taxiways, and contains aircraft navigational aids and a windsock in its center. The vegetation consists of native grasses that are mowed on a regular basis to a height of less than 15 cm (0.5 ft). The terrain is relatively flat, but is marked by irregularly spaced depressions less than 30 cm (1 ft) deep. Approximately 50 m

(160 ft) southwest from the windsock, a drainage ditch is terminated in a culvert. Due to its close proximity to active flight operations, access to the site is tightly controlled.

The primary contaminated area identified and described in the characterization survey report (Rademacher and Hoak 2000) was not in the immediate location of the aircraft crash site. The primary contaminated area identified and remediated was about 50 m (160 ft) from the crash site. This area corresponds to a shallow trench that was used to assist in extinguishing a persistent fire that engulfed one of the weapons. The only radioactive contaminant identified was depleted uranium (DU). During remediation, many small and one large DU slag were uncovered in a highly localized area that previously generated the highest in-situ γ-measurements and soil sampling results. The large slag appeared to be fused with small pebbles and sand, consistent with the historical record of the accident. This location is referred to as the "hot-spot" in other portions of the report.

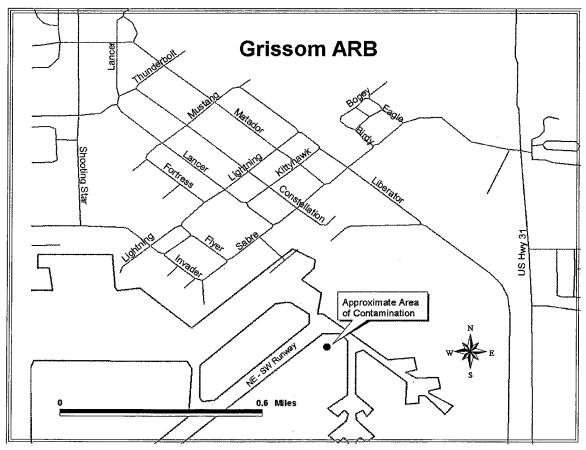


Figure 1: Site Location and Surrounding Area.

c. Summary of Completed Actions. The actions were completed in five phases. The preremediation site survey was accomplished 1 August 2000. The survey consisted of 330 in-situ γ -radiation measurements and soil samples for waste profiling. Soil removal operations were accomplished from 12 – 14 September 2000. Post removal, the area was scanned to locate any potential areas of residual contamination. A detailed remedial action support survey (RASS) was accomplished with fixed in-situ γ -measurements of the remediated area on 14 September 2000. Soil sampling was accomplished in a region equally comprised of the remediated area and immediate surrounding area. Soil sampling results and the RASS support the final site status. Clean soil was backfilled in the remediation area on 15 September 2000 and seeded for grass.

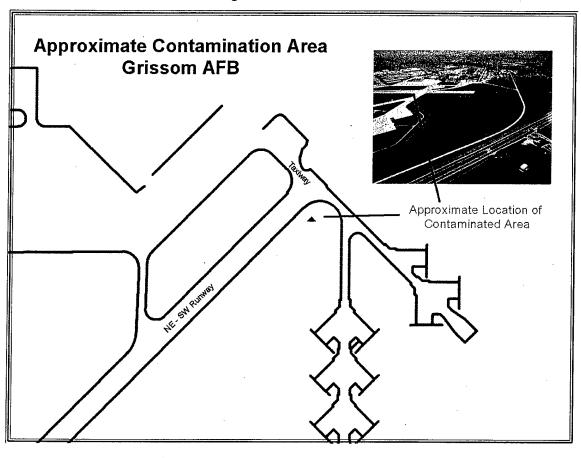


Figure 2: Site Location.

d. Summary of Final Site Status. In support of final site status determination, 25 soil samples were collected on a triangular grid pattern that encompassed the remediation area and an equal surrounding area for a total of 600 m^2 (6,500 ft²). Based on laboratory analysis, the mean residual DU on the site is estimated at 1.3 pCi g⁻¹. Based on this activity concentration level, the calculated dose equivalent level for hypothetical future site residents had a maximum of 0.2 mrem y⁻¹. This level is well below the 15 mrem y⁻¹ criterion recommended by the EPA.

2. Historical Site Assessment

a. Historical Record of Accident.

On 8 December 1964, during a routine Operational Readiness Inspection (ORI), a B-58 strategic bomber skidded off the runway at Bunker Hill AFB IN (later renamed Grissom AFB). The landing gear subsequently collapsed, rupturing a fuel tank. The ensuing fire burnt portions of the five nuclear weapons on board the aircraft. The high explosives in the weapons did not detonate, although some portions melted and burned (Sandia 1997). One burning weapon was removed from the accident area and extinguished by placing it in a shallow trench and covering it with sand. The trench was approximately 50 m (160 ft) from the aircraft wreckage in the grassy area between the runway and alert area taxiway. The historical record indicates radioactive contamination was confined to a 2 m x 6 m x 0.1 m (7 ft x 20 ft x 0.3 ft) volume (HQ Air Force Safety Center 1996). The precise location of the trench is unknown from the historical record. The contaminated soil around the aircraft wreckage was excavated and buried along with the aircraft debris at a different location on the base that now is the responsibility of the AFBCA (outside 434 ARW property). Extensive sampling of the area soon after the accident supported the claim that the area was contamination free. However, written documentation of post accident sampling has not been located. Additionally, instrumentation available at the time of the accident (primarily alpha scintillation and Geiger-Mueller detectors) coupled with the wet conditions questions sufficiency of detection of residual contamination with respect to present standards.

The recovered weapons and weapons debris were sent to Atomic Energy Commission facilities in Clarksville TN, Medina Base TX, Rocky Flats CO, Miamisburg OH, and Oakridge TN (Sandia 1997). Subsequent analysis of the damaged weapons and debris indicated that plutonium was not released to the environment because all of the plutonium-bearing components were intact (Rademacher 1999a).

In June of 1996, the HQ Air Force Safety Center, at the request of Grissom ARB, conducted a review of classified and unclassified documents in its possession and concluded that sufficient data did not exist to support unrestricted release of the site (HQ Air Force Safety Center 1996).

b. Scoping Surveys. In 1996, the IDH performed γ -radiation exposure rate measurements and collected soil samples from the accident site. The IDH identified a small area with γ -radiation exposure rates 8 to 10 times background rates. Soil samples collected from this area contained

concentrations that were several hundred times background for ²³⁸U. ²³⁵U and ²³⁴U concentrations were also elevated in proportion to that of depleted uranium, with the highest total uranium activity concentration over 700 pCi g⁻¹. Plutonium concentrations were consistent with levels typical of the background (EPA 1996). AFIERA conducted a scoping survey in May 1999 and collected five soil samples that had the isotopic composition of depleted uranium, with the highest total uranium activity concentration being 95 pCi g⁻¹.

- c. Characterization Survey. AFIERA performed a comprehensive site characterization in October 1999 that consisted of fixed and scanning in-situ y-measurements, and extensive soil sampling (Rademacher and Hoak 2000). An 8,800 m² (95,000 ft²) land area in square grids of 100 m² (1,100 ft²) was evaluated to determine background radiation levels; identify areas of elevated in-situ γ -radiation levels; and quantify uranium, americium, and chemical concentrations in soils. The scanning survey identified only one area of elevated contamination as shown by the pink box on the site grid of Figure 3. The pink box roughly encompassed 300 m² (3,200 ft²) and had an estimated mean excess total uranium (i.e., not attributed to natural background) surface activity concentration of 15 to 20 pCi g-1 (Rademacher and Hoak 2000). Figure 4 provides a more detailed map of the contaminated area with notation of the mean surface soils sampling results (234Th) for each grid and fixed in-situ γ -measurements in the hot-spot area. ²³⁴Th is typically used to quantify uranium because it is in the decay chain of 238 U and has an abundant γ -radiation emission. The pink rectangular box is similar in placement to the area of elevated concentration in Figure 3. For the 10 grids with mean surface ²³⁴Th activity concentrations greater than 2 pCi g⁻¹ (about two times natural background), the mean excess uranium activity concentration was about 7 pCi g⁻¹. Chemical analysis of targeted soil samples had beryllium concentrations typical for background. ²⁴¹Am activity concentrations were below the minimal detectable concentration (MDC) [ranged from 0.03 to 0.21 pCi g⁻¹] for the γ-spectroscopy system used. ²⁴¹Am is a daughter of ²⁴¹Pu and a co-contaminant of weapons grade plutonium (WGP). Based on other sites contaminated with WGP from the time period of this accident, the ^{239/240}Pu to ²⁴¹Am activity concentration ratio is about 5.4 (Rademacher 1999b), making sample ^{239/240}Pu activity concentrations less than 1.1 pCi g⁻¹.
- d. Contaminants of Concern. Based on the results of the AFIERA characterization study, the only contaminant of concern is depleted uranium (natural uranium depleted in ²³⁵U and ²³⁴U isotopes). Table 1 contains the isotopic composition of natural and depleted uranium. Uranium, a naturally occurring radioactive element, is silver-white in its pure form. It is a heavy metal nearly

twice as dense as lead (19 g cm⁻³). Uranium occurs in nature in a wide variety of solid, liquid, and gaseous compounds. It readily combines with other elements to form uranium oxides, silicates, carbonates and hydroxides. These compounds range from the highly mobile (e.g., carbonates) to the relatively immobile oxides. Hot-fired uranium in oxygen environments readily forms the oxides UO_2 , U_3O_8 , and U_2O_7 .

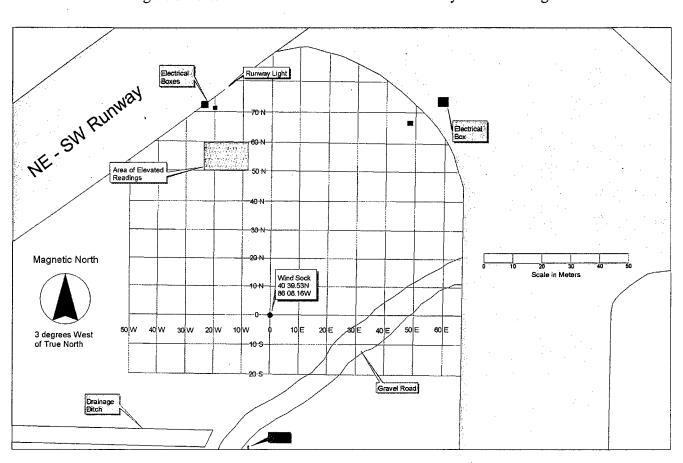
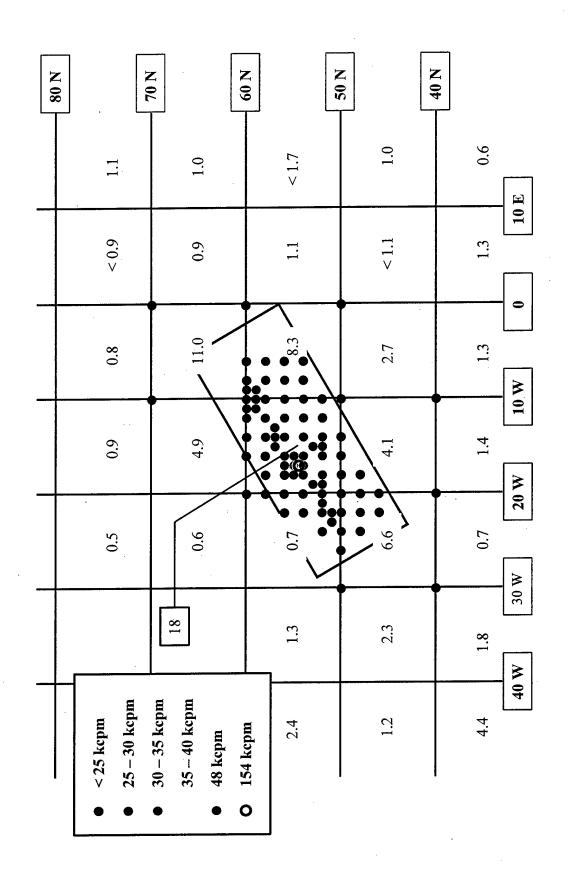


Figure 3. Site with Contaminated Area Denoted by Pink Rectangular Box.

Table 1. Isotopic Ratios of Natural and Depleted Uranium (DU).

	Component by Weight Percentage				Specific
Material	²³⁴ U	²³⁵ U	²³⁶ U	²³⁸ U	Activity (μCi g ⁻¹)
Natural U	0.0057%	0.72%	0%	99.28%	0.7
Depleted Uranium	0.0001%	0.20%	0.0003%	99.8%	0.4

Figure 4. Fixed In-Situ Gamma Measurement Results for Hot-Spot Area and Accompanying Mean Grid Surface Soil Sampling Results for ²³⁴Th (pCi g⁻¹) – Grid in Meters (Rademacher and Hoak 2000).



3. Remediation Goals

The characterization report contains dose estimates for hypothetical residents inhabiting the site in a contaminated condition (Rademacher and Hoak 2000). Hypothetical residents on the site are assumed to consume food grown on the land and water from an underground aquifer. Dose estimates were calculated with RESRAD Version 5.82 (Yu, *et al* 1993), a computer code developed at Argonne National Laboratory. The code accounts for radiation exposure from a multitude of routes including external γ -radiation, food and water consumption, and inhalation of resuspended soil particles. The code converts residual radioactive material activity concentrations in soils (expressed in pCi g⁻¹) into projected annual dose equivalent (expressed in mrem y⁻¹).

Table 2 contains a summary of the RESRAD calculations from the characterization report. The fourth data row contains an estimate of annual dose equivalent under the worst case assumption that all of the contaminant is concentrated in a 300 m² area. The value of 5.4 mrem y⁻¹ is a small fraction of 300 mrem y⁻¹, the average background radiation dose equivalent Americans receive, and below remediation criteria provided by regulatory agencies for exposures to the general public.

Table 2. RESRAD Dose Estimates with Various Parameters and Assumptions (Rademacher and Hoak 2000).

Area	Contamination	Length of	DU Activity	Years Post	Dose
(m^2)	Zone	Contamination	Concentration	Deposition	Equivalent
(111)	Thickness (m)	Zone (m)	(pCi g ⁻¹)	Dose (max)	(mrem y ⁻¹)
10,000	2	100	1	815	1.0
1,000	0.5	50	20	745	5.2
1,000	0.5	50	7	745	1.8
300	0.5	25	23.3	457	5.4

The depleted uranium contaminant is a Section 91b material as defined by the Atomic Energy Act of 1954. The Department of Defense (DoD) retains regulatory authority for the contaminant. For the Department of the Air Force, responsibility is delegated to Headquarters, Air Force Safety Center as provided in AF Instruction 40-201, *Managing Radioactive Materials in the Air Force*. The DoD has been directed to apply current accepted practice in maintenance and disposition of contaminated sites. Two Federal agencies have issued regulatory and proposed standards for residual radioactive materials. The Nuclear Regulatory Commission (NRC) has a risk-based regulatory standard for residual radioactive materials of 25 mrem y⁻¹, with the "as low as is reasonably achievable"

(ALARA) principle [10 Code of Federal Regulations (CFR), Part 20, Subpart E, Radiological Criteria for License Termination, 10 CFR 20.1402]. The Environmental Protection Agency (EPA) proposed a risk-based standard of 15 mrem y⁻¹, also with the ALARA provision (40 CFR 195, Radiation Site Cleanup Regulations).

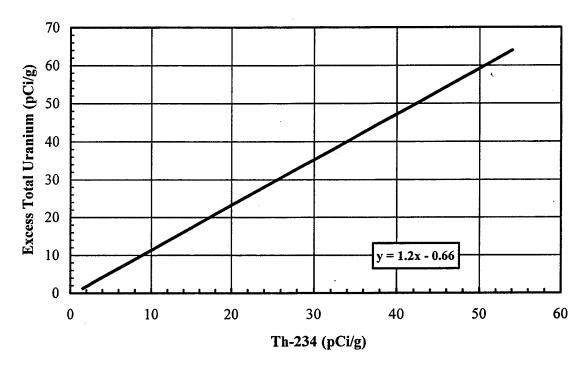
The 434th ARW made the decision to remediate the site based on the ALARA principle, though the site met both NRC and EPA primary criteria. AFIERA recommended delineation of the remediation area to locations of residual DU readily differentiated from background with portable instruments (Rademacher 2000). Portable 3 x 3 NaI(Tl) detectors were estimated to be sensitive to 6.4 pCi g⁻¹ of residual DU (Rademacher 2000). The IDH and EPA accepted this approach as protective of health and the ALARA principle.

4. Survey Methodology

- a. General. The survey accomplished in the pre-remediation phase delineated contaminated areas from background regions using portable NaI(Tl) detection systems. Areas with low levels of depleted uranium contamination that were not readily differentiated from background radiation areas were left as residual. Based on the background assessment for the site and the measured sensitivity of NaI(Tl) detectors to depleted uranium, total excess uranium activity concentrations of 6.4 pCi g⁻¹ could be discriminated from background sources of radiation (Rademacher 2000). The use of portable instruments reasonably balanced cost of contaminated waste disposal, survey time, equipment and personnel costs, and mobilization costs. Since health risks to employees was not determined to be significant, this factor was not incorporated into the evaluation. Use of portable instruments to assess soil removal requirements was highly advantageous over soil sampling and subsequent laboratory analysis because of the time lag between collection and sample analysis.
- b. In-Situ Measurements. Fixed in-situ measurements were collected on a two meter (6.6 ft) grid with a NaI(Tl) detection system. For the pre-remediation survey, plans were made to accomplish the survey with a 3 x 3 NaI(Tl) detector. However, early in the grid survey, the detector fell on its side, initiating a quality assurance/quality control (QA/QC) check to ensure proper operation. The instrument failed the chi-square tests for reliability and failed to provide similar background location measurements. As a result, the instrument was removed from service and replaced with a backup 2 x 2 NaI(Tl) detector. For the RASS, a 3 x 3 NaI(Tl) detector was used for scanning and fixed measurements. Appendix A contains a listing of the survey instruments.

- c. Soil Sampling of Remediation Zone for Waste Characterization. Based on the results of the pre-remediation survey, 10 soil samples were collected from the areas planned for remediation. The samples were analyzed for radionuclide content by γ -spectroscopy analysis. From 5 of the 10 locations, additional samples were collected for chemical analysis solely to meet disposal acceptance requirements of the waste disposal facility. Hazardous chemicals are not contaminants of concern.
- d. Post-Remediation Fixed In-Situ γ -Radiation Survey. Post-remediation fixed in-situ γ -radiation surveys were accomplished with a 3 x 3 NaI(Tl) on a two meter grid in the remediation zone. Based on the results of the survey, and through consultation with the 434th Environmental Engineering Flight (434th SPTG/CEV) and IDH, an additional 30 cm (12 in) lift was removed in a small area of the remediation zone. In-situ measurements were recollected on sampling locations in the area of additional remediation.
- e. Final Status Soil Sampling. A sampling grid was developed using the reference grid system implemented in the characterization study (Rademacher and Hoak 2000). The sampling grid encompassed the entire remediation area and an approximately equal area surrounding it. Twenty-five surface soil samples were collected using a triangular grid system as described in the Multi-Agency Radiation Survey and Site Investigation Manual (MARSSIM) (NRC 1997). Sampling depth was approximately 15 cm (0.5 ft) with each sample comprising at least one kilogram (2.2 lbs). The samples were collected with a small shovel. Decontamination was done between each sample using detergent and distilled water. Samples were double-wrapped in plastic bags. Chain of custody was documented. To maintain chain of custody, all samples were under constant observation or secured. All sample labels were completed using waterproof ink. For 10 of the sampling locations, replicate samples were collected and packaged in a similar manner and sent to Duke Engineering Services for analysis. Three replicates were collected for internal laboratory QA/QC. Samples were dried, homogenized, and analyzed through γ-spectroscopy. Estimated excess total uranium was calculated with ²³⁴Th activities and the relationship as shown in Figure 5.
- f. Field Contamination Control Assurance. To assure contamination was contained on the site, personnel and equipment leaving the remediation zone were screened with portable β/γ -radiation detection instruments as listed in Appendix A. A 3 x 3 NaI(Tl) was used to screen the backfilled area, the remediation support zone, and ingress/egress routes to ensure no contaminated soil was spread on the site.

Figure 5. Excess Activity Concentration of Excess Total Uranium vs. Measured ²³⁴Th for Background Total Uranium = 1.1 pCi g⁻¹ and a Depleted Uranium Contaminant.



g. Survey Personnel. Tables 3a and 3b contain a listing of survey personnel.

Table 3a. Survey Team Personnel (Pre-Remediation Survey).

Name	Position	Organization
Major Steven Rademacher	AFIERA Survey Chief	AFIERA/SDR, Brooks AFB TX
Mr. Brian Renaghan	Health Physicist	AFIERA/SDRH, Brooks AFB TX
SSgt Darrin Lawrence	Radioanalytical Technician/Sample Control	AFIERA/SDRD, Brooks AFB TX

Table 3b. Survey Team Personnel (Remediation Surveys).

Name	Position	Organization
Mr. Rex Bowser	Radiation Safety Inspector	Indiana Dept. of Health
Major Steven Rademacher	AFIERA Survey Chief	AFIERA/SDR, Brooks AFB TX
Mr. Brian Renaghan	Health Physicist	AFIERA/SDRH, Brooks AFB TX
SrA Kimberly Murchison	Bioenvironmental Engineering Technician	AFIERA/SDRH, Brooks AFB TX

5. Pre-Remediation Survey Results

- a. General. The pre-remediation survey was accomplished 1 August 2000. The grid system implemented in the characterization survey was recreated by locating the hot-spot and using a transit to align the grid. The 3 x 3 NaI(Tl) detector prepared for measurements fell and was removed from service. The 2 x 2 NaI(Tl) backup probe was used. Instrument background, QA/QC, and hot-spot measurements with this instrument are contained in Table B-1 of Appendix B. Fifteen background locations were marked in an area more than 50 m (160 ft) from the contamination zone. Ten measurements were taken at background location #1 to assess detection system reliability. Summary statistics of the background measurements are contained in Table B-2. For pre- and post-survey measurements at background location #1, the instrument was assessed to be reliable due to the close agreement in mean count rate of each data set (RPD = 1.6 %) and the calculated percent coefficient of variation (% CV) of each data set with the theoretical expectation of about 1.6 %.
- b. Grid Measurements. Grid measurements were collected every two meters, with the detector to ground surface separation of 7.5 cm (3 in). The measurements ranged from 3,524 to 5,794 counts per 30-second count period, with background at 4,097 counts. Table B-3 contains a complete listing of grid measurements. Figure B-1 provides a graphical depiction of the grid measurements. The plot, comprised of 333 measurements, provides more detail than that of the characterization survey (Figure 4) but is consistent with previously identified contaminated areas.
- c. Correlation Coefficients. Correlation between the grid measurements of this survey and that of the characterization survey was assessed to determine relative sensitivity of the 2 x 2 NaI(Tl) and 3 x 3 NaI(Tl). Figure B-2 contains a scatterplot of the data for paired grid measurements. The slope of the regression analysis is 0.187. Modified to account for the different integration times, the slope is 0.374, with the 3 x 3 NaI(Tl) being 2.67 times more efficient. The squared correlation coefficient of 0.87 is reasonable, considering that some minor differences may have existed in the two grid systems and instrument response fluctuation due to ambient temperature variation.
- d. Waste Profiling. Soil samples were collected uniformly within the contamination zones. The sampling was accomplished for pre-profiling the waste for eventual disposal. Figure B-3 contains a scatterplot of the soil ²³⁴Th activity concentrations vs. the paired instrument measurement. The slope of the regression is 26.4 counts g pCi⁻¹, with good correlation between the two parameters, demonstrating that in-situ measurements provide good surrogates to laboratory analysis of soil

samples. Table B-4 contains the soil sampling results for ²³⁴Th, ²³⁵U, and ²⁴¹Am for the specified sampling locations. The mean ²³⁴Th activity concentration among the samples was 14.2 pCi g⁻¹, with the estimated mean excess DU activity of 16 pCi g⁻¹ using the relationship in Figure 5. This estimate is within the range estimated in the characterization report of 15 – 20 pCi g⁻¹. Table B-5 contains the toxicity characteristic leaching procedure (TCLP), metals, and herbicide analytical results for the five waste profiling samples. The results verify that the waste did not contain hazardous chemical contaminants that would restrict disposal as a radioactive waste.

6. Remediation Activities and Post-Remediation Survey Results

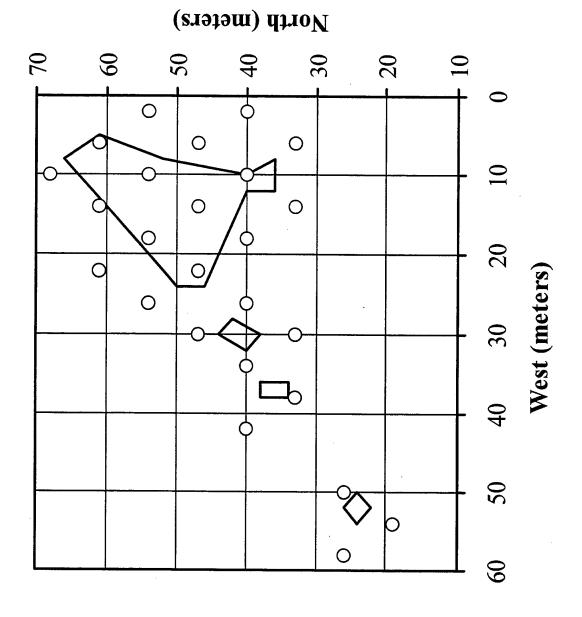
- a. Remediation Zone Delineation. The remediation zone was marked 11 September 2000, the second phase of the project. Figure 6 graphically depicts the zone by the solid black line, with final site status soil sampling locations denoted by the circles. The zone was created by evaluation of Figure B-1, with respect to detector response. The upper limit of the violet contour corresponds to DU activity concentrations of about 6.4 pCi g⁻¹, based on a mean background count rate of 4,097 counts per 30-second and a detector to ²³⁴Th contaminant correlation coefficient of 26 counts g pCi⁻¹.
- b. Background Measurements. 3 x 3 NaI(Tl) instruments were used to collect background measurements at the same location as that of the pre-remediation survey. Multiple measurements were collected to assess instrument reliability. Background measurements were collected every day, with multiple assessments on 14 September 2000 because RASS measurements were collected. Tables C-1a C-1j contain background measurement summaries and associated theoretical and calculated standard deviations. Based on comparisons of theoretical and calculated standard deviations, survey instruments were determined to be reliable. Measurements were also collected at the hot-spot location to verify that survey instruments had a response similar in proportion to that noted from the pre-remediation survey (Tables C-2a C-2c).
- c. First Soil Removal Operation. Soil removal operations in the delineated zone were initiated on 12 September 2000. Due to rain, removal of the top 30 cm (12 in) was not complete until 14 September. The operation filled 12 containers with a disposal volume of 87 m³ (3,100 ft³).
- d. Remedial Action Support Survey. After the first soil removal operation, a scanning survey of the remediation areas was completed. Areas of highly elevated activity were not located during the scanning survey. The 2.0 m (6.6 ft) grid was re-established in the remediation areas.

Figure 6. Remediation
Zone Delineation and
Final Status Soil
Sampling
Locations

- Delineation Line

Soil Sampling Locations

0



Fixed measurements were collected in the remediation area. For the small excavated areas, the grid had a limited number of survey points, requiring some off-grid measurements to ensure a sufficient number of locations were surveyed. In the former hot-spot area of the large excavation area, off-grid measurements were collected to more finely investigate localized contamination. Table C-3 contains a complete listing of the measurements. The 72 measurements ranged from 10,722 to 12,640 counts. For the excavated areas, measurements were expected to be slightly elevated over those measurements from the unremediated area for two reasons. First, the grass zone provides some attenuation of the photons emitted from the DU. Second, the geometry of the two measurement conditions is different. In excavated areas, a detector will have more γ -radiation entering the detector side than in level-grade areas. Overall, these two factors were estimated to increase detector response by 250 counts [30 s] in the excavated areas. A graphical display of the survey data is in Figure C-1, with the omission of data collected off the 2.0 m grid. The upper limit of the violet contour corresponds to DU contamination of about 8 pCi g⁻¹. [Note. The violet contours of Figures B-1 and C-1 represent different count ranges and estimated ²³⁴Th activity concentrations.]

- e. Second Soil Removal Operation. After reviewing the data in Figure C-1, Major Rademacher, Mr. Renaghan, and Mr. Bowser decided to remove another 30 cm (12 in) from the localized area of elevated contamination of the large excavation area. Appendix E contains copies of the waste shipment manifest. Some elevated measurements existed in two of the small excavated areas, but were deemed largely to be the result of geometry (i.e., a detector edge effect). It was agreed that additional remediation in these areas was unnecessary.
- f. Final Site Status Survey. After the second soil removal operation was complete, the 2.0 m (6.6 ft) grid was re-established in the excavated area. Measurements were collected and are listed in Table C-4, with a listing of measurements collected prior to the second soil removal operation. Evaluation of the seven paired-measurements showed that five of the repeated measurements were lower, while two were increased. The five locations that had reduced measurements represented those locations with the highest initial measurement; leading to the conclusion that the second soil removal operation was beneficial. The two locations with an increased measurement were close to the wall of the excavation, with the increased count rate assumed to be the result of the detector edge effect. A graphical display of the final site status data is in Figure C-2. An agreement was made between Major Rademacher, Ms. Marx, and Mr. Bowser that additional soil removal was not necessary because the area with residual contamination (differentiable from background) was limited

and had an estimated activity concentration near the final screening criterion of 8 pCi g⁻¹ (Rademacher 2000). Final status soil samples were then collected based on the grid in Figure 6.

g. Final Site Status Soil Sampling Results. The results of the final site status soil samples are listed in Table C-6 for ²³⁵U, ²³⁴Th, and ²⁴¹Am. For archival purposes, complete analytical results for the private laboratory are provided in Appendix D. The ²³⁴Th activity concentrations are assumed to be in secular equilibrium with the parent ²³⁸U and can be directly used to estimate the residual DU according to the relationship in Figure 5. The ²³⁵U and ²⁴¹Am are listed to confirm that the soils do not contain enriched uranium and WGP contamination, respectfully. The results for ²³⁴Th ranged from < 0.5 to 7.4 pCi g⁻¹, with an average of 1.7 pCi g⁻¹ (under the assumption that the less than value data points were at the maximum). Using the relationship in Figure 5, the estimated average residual DU is 1.3 pCi g⁻¹ (95 % confidence interval: 0.7 to 2.1 pCi g⁻¹). The ²⁴¹Am activity concentrations were typical for background, supporting the conclusion that WGP is not a residual soil contaminant. For all samples except GS0000276, the one with highest ²³⁴Th activity concentration, the ²³⁵U activity concentrations are typical for background, supporting the conclusion that enriched uranium is not a residual soil contaminant. The ²³⁴Th activity concentration value of 7.4 pCi g⁻¹ was the highest. It corresponds to an estimated DU activity concentration of 8.2 pCi g⁻¹ and was collected in the small area of the large remediation area where final fixed y-radiation measurements were the highest. The value of 8.2 pCi g⁻¹ corresponds well to the post-remediation γ radiation screening criterion of 8 pCi g⁻¹, validating the survey methodology. The ²³⁵U activity concentration for this sample is consistent for DU and is not indicative of an enriched uranium contaminant.

7. Final Status RESRAD Calculations

RESRAD Version 5.82 (Yu *etal* 1993) was used to estimate radiological exposure conditions existing on the site, under the assumption that the site was occupied by residents that derived agricultural benefits from the land. This exposure assumption is believed to be the most restrictive (i.e., worst case exposure).

Default parameters were used for the calculations with the following site conditions: DU activity concentration = $1.3 \, \text{pCi g}^{-1}$ (isotopic mix of Table 1), contamination zone thickness = $0.3 \, \text{m}$, contaminated area = $600 \, \text{m}^2$, length parallel to aquifer = $24 \, \text{m}$, and no cover over the residual contamination zone. The "no cover" assumption is conservative and provides an estimated dose

equivalent to future residents higher than that of actual site conditions. The residual contamination zone was covered with 0.5 m (1.5 ft) of clean soil.

The estimated current dose equivalent rate for the site is 0.11 mrem yr⁻¹, with about 75 % of the dose equivalent from direct external γ -radiation (ground). The maximum dose equivalent rate for the site, 0.2 mrem y⁻¹, is projected 700 y from now, with most of the dose equivalent from water pathways. The current and maximum projected dose equivalent rates for the site are respectively 136 and 75-fold lower than the EPA recommended residential dose equivalent rate of 15 mrem y⁻¹ and significantly lower than that estimated before remediation (Table 2).

8. Quality Assurance/Quality Control

- a. General. A number of measures were taken to ensure that the data produced could be used to adequately assess the effectiveness of the remediation and the final site status assessment.
- b. Soils. Three soil sampling locations had AFIERA Radioanalytical Laboratory replicate sample analysis, while at 10 locations similar replicates were field split (split replicates) and analyzed by AFIERA and Duke Engineering Services. Table 4 contains paired replicate sample mean ²³⁴Th activity concentrations, relative percent difference (RPD) values, and theoretical RPD values expressed at the 50 and 90 % confidence interval (CI). The RPD for low activity

Table 4. Calculated and Absolute Theoretical RPDs for ²³⁴Th in Replicate Soil Samples.

Sample	Mean	Calculated	Theoretical A	Absolute RPD
Location	²³⁴ Th (pCi g ⁻¹)	RPD	50 % CI	90 % CI
26N - 50W	3.7	7	30	78
33N – 14W	0.8	73	69	255
33N - 30W	0.7	56	27	70
33N – 38W	1.6	- 19	32	86
40N – 2W	1.2	3	35	98
40N – 10W	1.6	7	16	40
40N – 26W	0.9	41	40	110
47N – 14W	4.0	15	33	83
54N – 2W	1.9	42	19	47
54N – 18W	6.6	- 23	10	24
54N – 26W	0.7	- 44	48	147
61N – 6W	3.4	4	8	21
61N – 14W	1.2	31	41	127

concentration samples was significantly higher than that of the higher activity concentration samples. For paired samples with mean ²³⁴Th activity concentrations less than 1.2 pCi g⁻¹, the RPD ranged from 3 to 73 %. For samples with higher ²³⁴Th activity concentration, the absolute RPD ranged from 4 to 42 %. Eight of the 13 replicates had RPD values within the 50 % CI, while all were within the 90 % CI.

c. Fixed In-Situ γ-Measurements. Ten locations had replicate in-situ measurements. The purpose of the measurements was to evaluate variability in field measurements. Table C-5 contains a summary of results. The difference values among the measurements ranged from –211 to 764, with a mean of 280 counts. The difference between the two sets of measurements is beyond that anticipated for random counting uncertainties alone (95 % confidence interval: ± 200 counts). Since the measurements were collected over a period of a few hours, and there were significant changes in ambient temperature and sun exposure during the day, the variability observed is reasonable for portable NaI(Tl) systems.

9. Conclusions

The remedial actions of the contaminated areas were accomplished according to the workplan with about 87 m³ (3,100 ft³) DU contaminated soils disposed at Waste Controls Specialists, LLC, a company licensed to operate a low-level radioactive waste disposal facility. Field measurements and laboratory analysis of post-remediation soil samples confirm that the goals of the remediation effort were achieved. The mean activity concentration of residual DU contaminant was estimated at about 1.3 pCi g⁻¹ [95% CI: 0.7 to 2.1 pCi g⁻¹]. Calculated dose equivalent rates for hypothetical future site residents had a maximum of 0.2 mrem y⁻¹. This level is well below the 15 mrem y⁻¹ criterion that was recommended by the EPA. Although the installation has no present interest in utilization of the site for other purposes, the site meets the unrestricted public use criterion with respect to radiological concerns. No further remedial actions are recommended for the site.

10. References

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Appendix A Survey Instrumentation

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Table A. AFIERA Survey Instrumentation.

			Estimated Minimal
Measurement Type	Location	Instrumentation	Detectable
			Concentration
In-situ Gamma	Remediation Area -	Eberline Model SPA-3	20 pCi g ⁻¹ DU
(scanning + fixed)	7.5 cm above surface	2 x 2 NaI (Tl) Detector	(scanning)
		w/ Ludlum 2221	
		Ratemeter/Scaler	6.4 pCi g ⁻¹ DU (fixed)
		Ludlum Model 44-20	
		3 x 3 NaI (Tl) Detector	
		w/ Ludlum 2221	
		Ratemeter/Scaler	
Gamma Exposure Rate	Remediation Area	Ludlum Model 19	10 μR hr ⁻¹
		Exposure Rate	
		SN 138433	
Beta/Gamma Frisker	Remediation Area,	Bicron Analyst	300 dpm (Sr-90)
	Personnel, Equipment	SN A505P w/	
		Bicron PGM	
		SN C702B	
	·	Bicron Surveyor	
		SN A809P w/	
		Bicron PGM	
		SN C714B	

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Appendix B

Pre-Remediation Survey Data

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Table B-1. Instrument Background and Quality Assurance/Quality Control Measurements.

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	4,046	1240	759	6.1
Background #1	4,010			
Background #1	4,079			
Background #1	4,088			
Background #1	4,100			
Background #1	4,077			
Background #1	4,172			
Background #1	4,207			
Background #1	4,099			
Background #1	4,091			
Background #2	4,130			
Background #3	4,030			
Background #4	4,193			
Background #5	4,128			
Background #6	4,023			
Background #7	4,168			
Background #8	4,219			
Background #9	4,056			
Background #10	4,156			
Background #11	3,928			
Background #12	4,037			
Background #13	4,119			
Background #14	4,020			
Background #15	4,084			
55N – 17W	21,868	1320		
55N – 17W	22,047			
55N – 17W	21,822	1440	759	6.0
55N – 17W	21,095	1640	759	5.9
55N – 17W	21,543	1735	758	5.9
55N – 17W	22,114	1810	758	5.9

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Table B-1 (Continued). Instrument Background and Quality Assurance/Quality Control Measurements.

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	3,994	1810	758	5.9
Background #1	4,002			
Background #1	4,032			
Background #1	4,102			
Background #1	4,087			
Background #1	4,010			
Background #1	3,986			77/2
Background #1	3,943			
Background #1	4,010			
Background #1	4,147	1820		

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude)

B1 Grid Location: 10S - 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 132851

Meter Settings: HV = 759 V, Threshold = 119, Window = 3692 (operated in "OUT" position)

Probe: Ludlum SPA3 2 x 2 NaI(Tl), Serial Number 408881-002

Table B-2. Statistics for QA/QC Measurements.

Monovement Crove	Statis	tics (30 s Counting T	imes)
Measurement Group	μ	σ	% CV
Pre-Survey Background #1	4,097	57	1.4 %
Pre-Survey Backgrounds	4,089	79	1.9 %
Post-Survey Background #1	4,031	62	1.5 %

Table B-3. Pre-Remediation Grid Measurements (30 s Counts).

Location				Location	on North (meters)			
West (m)	20	22	.24	26	28	30	32	34	36
0									
2									
4									
6									
8									4141
10									4287
12									4035
14									4291
16									4111
18									
20									
22								4173	4220
24								4213	4253
26						,			4207
28									4157
30									4152
32									4114
34								4077	3978
36							3900	4320	4180
38						4001	4059	4039	4324
40			4081	4181	4121	4216	4117	4097	4099
42		4110	4231	4303	4290	4094	4233	4091	3986
44			4094	4213	4179	4193	4058	4006	3856
46	4018	4210	3947	4091	4172	4205	4165	3901	3884
48	4131	4029	3959	4082	4233	4153	3952	3770	3775
50	4154	4118	4068	4217	4218	4202	4078	3948	3762
52	3942	3907	4406	4022	3940	3864			
54		4241	4245	3997	4054	3675			
56			4091	3993	3783				
58			3675	3854					

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Table B-3 (Continued). Pre-Remediation Grid Measurements (30 s Counts).

Location				Locati	on North (meters)			
West (m)	38	40	42	44	46	48	50	52	54
0									
2			4068	3937	4154	4102	4204	4064	4185
4			4098	4067	4210	4045	4250	4274	4042
6	4131	4168	4030	4155	4177	4056	4126	4203	4012
8	4142	4000	4184	4055	4118	4136	4163	4233	4304
10	4312	4231	4435	4163	4348	4266	4428	4326	4489
12	4301	4234	4228	4337	4270	4110	4326	4411	4981
14	4198	4155	4321	4392	4253	4401	4219	4749	5430
16	4318	4199	4122	4104	4239	4385	4343	4634	5794
18	4071	4108	4156	4316	4033	4192	4169	4666	4839
20	4093	4117	4229	4274	4287	4259	4003	4605	4194
22	4311	4099	4141	4234	4258	4440	4588	4296	3997
24	4100	4140	4097	4209	4205	4157	4182	3937	3936
26	3968	4121	4073	4018	4161	4124	4106	4028	3946
28	4172	4062	4280	4148	4094	3947	3960	3878	
30	4214	4330	4324	4140	3997	3899	4073	3995	
32	4158	4269	4129	4029	4044	4017			
34	4034	4208	4170	4167	4077	3846			
36	4308	3999	4002	3995	4013	3860			
38	4186	4035	3965	3836	3924	3781			
40	4087	3898	3854	3865	3717				
42	3794	3762	3824						
44	3812								
46	3775								
48	3645								
50	3643								
52									
54									
56									
58									

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Table B-3 (Continued). Pre-Remediation Grid Measurements (30 s Counts).

Location			Location	on North (meters)		
West (m)	56	58	60	62	64	66	68
0		4009	4017	4012	3933	3896	3860
2	4095	4127	4053	4139	3988	3877	3947
4	4015	4218	4257	4034	3948	3922	3859
6	4222	4218	4107	4300	4194	3861	3805
8	4246	4403	4441	4559	4329	3860	3905
10	. 4859	5423	4472	4287	4158	4116	4004
12	5172	4797	4283	4011	4044	3893	3671
14	5661	4195	4220	4084	3926	3803	3524
16	4545	4086	4019	4024	3943	3717	3762
18	4169	4018	3795	3979	3947	3925	
20	3861	3884	3875	3933	3802		
22	3919	3854	3833	3690			
24	3848	3769					
26	3982	3743					
28							
30							
32							
34							
36							
38							
40							
42							
44							
46							
48							
50			-				
52							
54							
56							
58		× .					

Figure B-1. Pre-Remediation In-Situ γ -Radiation Measurements - (n = 333).

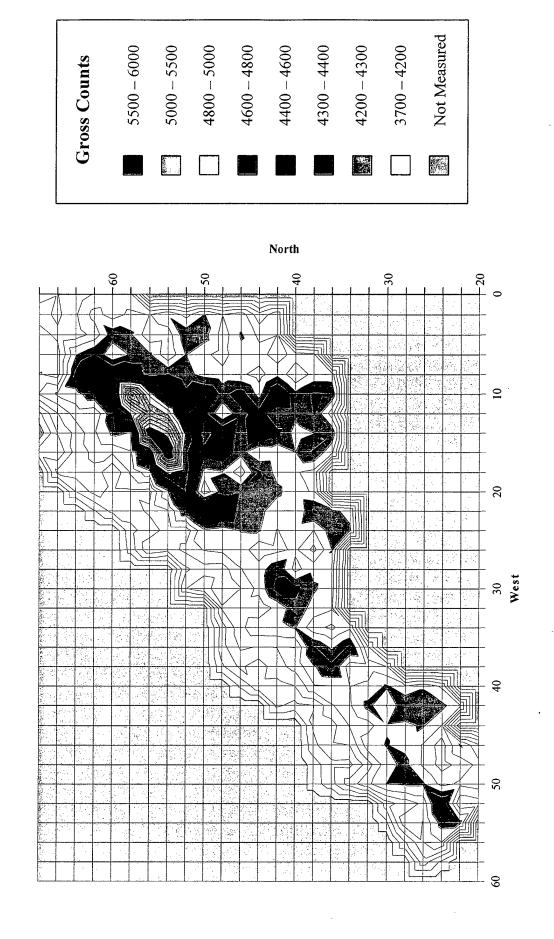


Figure B-2. Scatterplot of Characterization Survey 3 x 3 NaI(Tl) Measurements vs. Pre-Remediation Survey 2 x 2 NaI(TI) Measurements for Paired Grid Points.

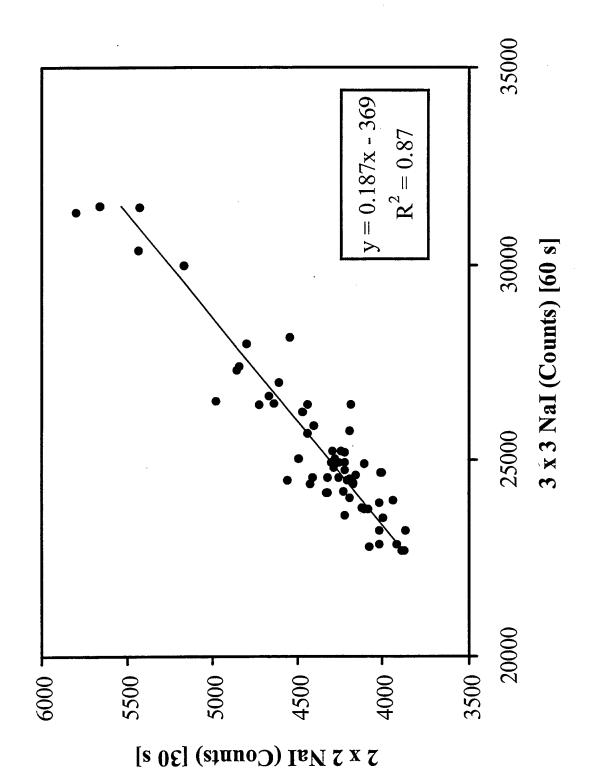


Figure B-3. Scatterplot of Pre-Remedial Action Survey 2 x 2 NaI(TI) Measurements vs. Paired Laboratory γ -Spectroscopy Analysis for 234 Th.

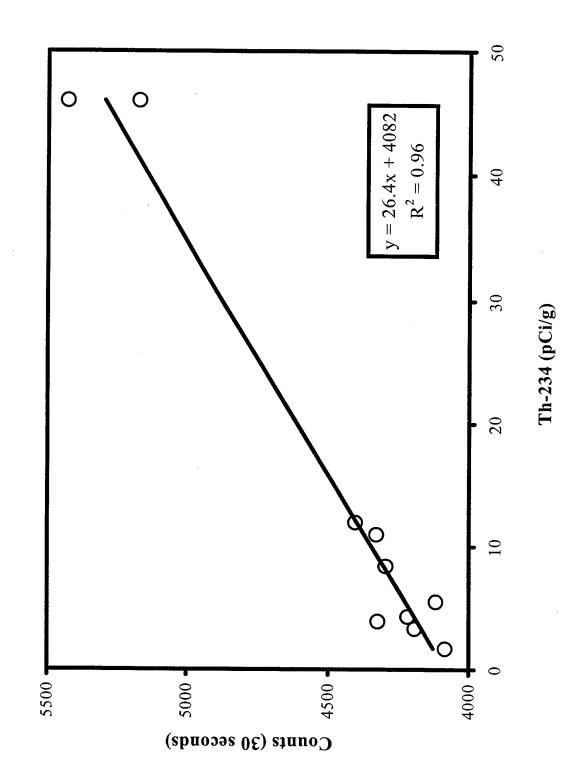


Table B-4. Pre-Remediation Soil Sample γ-Spectroscopy Results.

Sample	Base Sample	²³⁴ Th (pCi g ⁻¹)	²³⁵ U (p	oCi g ⁻¹)	²⁴¹ Am
Location	Number	Value	Uncertainty	Value	Uncertainty	(pCi g ⁻¹)
52N – 22W	GS0000552	8.4	0.9	0.27	0.14	< 0.13
42N – 30W	GS0000553	3.9	0.8	< 0	0.16	< 0.16
54N – 14W	GS0000554	46	3	< 0	0.22	< 0.2
54N – 20W	GS0000555	3.3	0.5	< (0.1	< 0.09
62N – 14W	GS0000556	1.7	0.6	< 0	0.09	< 0.09
40N – 20W	GS0000557	5.5	0.6	0.16 0.09		< 0.10
64N – 8W	GS0000558	10.7	0.8	0.22 0.11		< 0.11
58N – 8W	GS0000559	11.7	0.7	0.26	0.06	< 0.07
56N – 14W	GS0000560	46	2	0.81	0.10	< 0.11
50N – 14W	GS0000561	4.3	0.4	< 0	0.06	< 0.06
M	ean .	14.2*	5.7	0.2)**	0.11

^{*} Total Estimated Uranium Activity Concentration is 16.3 pCi g⁻¹ [Net = 15.2 pCi g⁻¹] using assumptions of Rademacher and Hoak (2000)

Uncertainty Values at the 1σ Level

^{**} Based on Estimated Ratio of U-238 to U-235 in DU using assumptions of Rademacher and Hoak (2000)

Table B-5. Pre-Remediation Soil Sample TCLP-Metals Results.

	EPA		- 64N	-8W	52N-	-38W	52N-	- 22W	40N -	-10W	26N-	- 52W
Analyte	Method	Units	Report Limit	Result								
Arsenic	6010B	mg L ⁻¹	0.25	< 0.25	0.25	< 0.25	0.25	< 0.25	0.25	< 0.25	0.25	< 0.25
Barium	6010B	mg L ⁻¹	0.005	2.37	0.005	2.79	0.005	2.08	0.005	1.92	0.005	2.07
Cadmium	6010B	mg L ⁻¹	0.005	<5E-3								
Chromium	6010B	mg L ⁻¹	0.01	<0.01	0.01	<0.01	0.01	<0.01	0.01	<0.01	0.01	<0.01
Lead	6010B	mg L ⁻¹	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5
Selenium	6010B	mg L ⁻¹	0.1	<0.1	0.1	<0.1	0.1	<0.1	0.1	<0.1	0.1	<0.1
Silver	6010B	mg L ⁻¹	0.005	0.082	0.005	900.0	0.005	0.01	0.005	0.118	0.005	<5E-3
1,1-Dichloroethane	8260B	mg L ⁻¹	0.7	<0.7	0.7	<0.7	0.7	<0.7	0.7	<0.7	0.7	<0.7
1,2-Dichloroethane	8260B	${ m mg}{ m L}^{{ m -}1}$	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5
1,4-Dichloroethane	8260B	mg L ⁻¹	7.5	<7.5	7.5	<7.5	7.5	<7.5	7.5	<7.5	7.5	<7.5
Benzene	8260B	mg L ⁻¹	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5
CCI4	8260B	mg L ⁻¹	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5
Chlorobenzene	8260B	${ m mg}{ m L}^{-1}$	100	<100	100	<100	100	<100	100	<100	100	<100
Chloroform	8260B	mg L ⁻¹	6.0	0.9>	0.9	0.9>	0.9	0.9>	0.9	<6.0	0.9	<6.0
MEK	8260B	mg L ⁻¹	200	<200	200	<200	200	<200	200	<200	200	<200
Tetrachloroethene	8260B	mg L ⁻¹	0.7	<0.7	0.7	<0.7	0.7	<0.7	0.7	<0.7	0.7	<0.7
Trichloroethylene	8260B	mg L ⁻¹	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5
Vinyl Chloride	8260B	mg L ⁻¹	0.2	<0.2	0.2	<0.2	0.2	<0.2	0.2	<0.2	0.2	<0.2
2,4,5-Trichlorophenol	8270C	${ m mg}{ m L}^{\text{-1}}$	400	<400	400	<400	400	<400	400	<400	400	<400
2,4,6-Trichlorophenol	8270C	mg L ⁻¹	2.0	<2.0	2.0	<2.0	2.0	<2.0	2.0	<2.0	2.0	<2.0
i												

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Table B-5. Pre-Remediation Soil Sample TCLP-Metals Results (Continued).

	EPA	,	- 64N	-8W	- N2S	38W	- 52N -	22W	40N -	10W	26N -	52W
Analyte	Method	Units	Report Limit	Result								
2,4-Dinitrotoluene	8270C	$mg L^{-1}$	0.13	<0.13	0.13	<0.13	0.13	<0.13	0.13	<0.13	0.13	<0.13
2-Methylphenol	8270C	mg L ⁻¹	200	<200	200	<200	200	<200	200	<200	200	<200
3-+4-Methylphenol	30728	mg L ⁻¹	200	<200	200	<200	200	<200	200	<200	200	<200
Chlordane	8270C	mg L-1	0.03	<0.03	0.03	<0.03	0.03	<0.03	0.03	<0.03	0.03	<0.03
Endrin	8270C	mg L-1	0.02	<0.02	0.02	<0.02	0.02	<0.02	0.02	<0.02	0.02	<0.02
Heptachlor	8270C	${ m mg}{ m L}^{ ext{-1}}$	0.008	<0.01	0.008	<0.01	0.008	<0.01	0.008	<0.01	0.008	<0.01
Heptachlor Epoxide	8270C	mg L ⁻¹	0.008	<0.01	0.008	<0.01	0.008	<0.01	0.008	<0.01	0.008	<0.01
Hexachlorobenzene	8270C	mg L ⁻¹	0.13	<0.13	0.13	<0.13	0.13	<0.13	0.13	<0.13	0.13	<0.13
Hexachlorobutadiene	8270C	mg L ⁻¹	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5	0.5	<0.5
Hexachloroethane	8270C	mg L ⁻¹	3.0	<3.0	3.0	<3.0	3.0	<3.0	3.0	<3.0	3.0	<3.0
Lindane	8270C	${ m mg}{ m L}^{\text{-1}}$	0.4	<0.4	0.4	<0.4	0.4	<0.4	0.4	<0.4	0.4	<0.4
Methoxychlor	8270C	${ m mg}{ m L}^{\text{-1}}$	10	<10	10	<10	10	<10	10	<10	10	<10
Nitrobenzene	8270C	${ m mgL^{-1}}$	2.0	<2.0	2.0	<2.0	2.0	<2.0	2.0	<2.0	2.0	<2.0
Pentachlorophenol	8270C	${ m mgL^{-1}}$	100	<100	100	<100	100	<100	100	<100	100	<100
Pyridine	8270C	${ m mgL^{-1}}$	5.0	<5.0	5.0	<5.0	5.0	<5.0	5.0	<5.0	5.0	<5.0
Toxaphene	8270C	${ m mg}{ m L}^{{ m -l}}$	0.5	<0.1	0.5	<0.1	0.5	<0.1	0.5	<0.1	0.5	<0.1
Reactivity Cyanide	9014	μg G ⁻¹	1.0	<1	1.0	<1	1.0	<1	1.0	<1	1.0	<1
Reactivity Sulfide	9034	μg G-1	12	<12	12	20	12	20	12	<12	12	<12

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Table B-5. Pre-Remediation Soil Sample TCLP-Metals Results (Continued).

,	EPA	j	- W49	64N - 8W	52N -	52N - 38W	52N -	52N - 22W	40N-	40N - 10W	26N –	- 52W
Analyte	Method	Units	Report Limit	Result	Report Limit	Result	Report Limit	Result	Report Limit	Result	Report Limit	Result
Hydrogen Ion	9040B	Hd		7.2	0.01	8.0	0.01	8.2	0.01	7.1	0.01	8.0
Mercury	SW7470 mg L ⁻¹	mg L ⁻¹	5E-3	<5E-3	5 E-3	<5E-3	5 E-3	<5E-3	5 E-3	<5E-3	5 E-3	<5E-3
2,4,5-TP (Silvex)	8151A mg L ⁻¹	mg L ⁻¹	NA	<1.0	NA	<1.0	NA	<1.0	NA	<1.0	NA	<1.0
2,4-D	8151A mg L ⁻¹	mg L ⁻¹	NA	<10	NA	<10	NA	<10	NA	<10	NA	<10

Appendix C

Remediation Phase Survey Data

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Table C-1a. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (11 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	10,016	1300	863	5.9
Background #2	9,881			
Background #3	9,856			
Background #4	9,943			
Background #5	9,855			
Background #6	10,040	`		
Background #7	10,067			
Background #8	9,895			
Background #9	9,947			
Background #10	10,023			
Background #11	9,860			
Background #12	10,036			
Background #13	9,858			
Background #14	9,825			
Background #15	9,843			
Background #16	9,908			
Background #17	9,967			
Background #18	10,023			
Background #19	9,818			
Background #20	9,749	1330	863	5.9

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 863 V, Threshold = 102, Window = 3840 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 9,920.5 counts Standard Deviation of Measurements: 90.6 counts

% CV: 0.9 %

Expected Standard Deviation: 99.6

Table C-1b. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #2] (11 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	10,031	1300	742	5.9
Background #2	9,897			
Background #3	9,940			
Background #4	9,820			
Background #5	9,771			
Background #6	9,924			
Background #7	9,989			
Background #8	10,046			
Background #9	9,862			***************************************
Background #10	9,862	10.10.11		
Background #11	9,932			
Background #12	9,880			
Background #13	9,716			
Background #14	10,011		,	
Background #15	9,792			
Background #16	9,753			
Background #17	9,938			
Background #18	9,879			
Background #19	9,949			
Background #20	10,008	1330	742	5.9

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 132849

Meter Settings: HV = 742 V, Threshold = 102, Window = 3840 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(T1), Serial Number 165386 Calibrated: 8 Sep 00

Mean Background Response: 9,900.0 counts Standard Deviation of Measurements: 95.0 counts

% CV: 0.96 %

Expected Standard Deviation: 99.5

Table C-1c. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (12 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	10,019	0935	867	6.0
Background #2	9,885			
Background #3	9,848			
Background #4	9,935			
Background #5	9,960			
Background #6	9,648			
Background #7	9,874			
Background #8	9,847			
Background #9	9,979			
Background #10	9,958			
Background #11	9,851			
Background #12	9,865			•
Background #13	10,120			
Background #14	10,077			
Background #15	9,901			
Background #16	9,760			
Background #17	9,933			
Background #18	9,743			
Background #19	9,892			
Background #20	9,899	1000	867	6.0

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 863 V, Threshold = 103, Window = 3851 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 9,899.7 counts Standard Deviation of Measurements: 109.4 counts

% CV: 1.1 %

Expected Standard Deviation: 99.5 counts

Table C-1d. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #2] (12 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	9,789	0935 743		5.8
Background #2	9,587			
Background #3	9,765			
Background #4	9,741			
Background #5	9,842			
Background #6	9,669			
Background #7	9,903			
Background #8	9,763			
Background #9	9,794			
Background #10	9,825			
Background #11	9,613		·	
Background #12	9,908			
Background #13	9,761			
Background #14	9,818			
Background #15	9,862			
Background #16	9,687			
Background #17	9,911			
Background #18	9,586			
Background #19	9,889			
Background #20	9,750	1000	743	5.8

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 132849

Meter Settings: HV = 743 V, Threshold = 101, Window = 3874 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 165386 Calibrated: 8 Sep 00

Mean Background Response: 9,773.2 counts
Standard Deviation of Measurements: 102.8 counts

% CV: 1.05 %

Expected Standard Deviation: 98.9 counts

Table C-1e. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (14 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	10,973	1100	869	5.9
Background #2	10,842			
Background #3	11,011			
Background #4	10,969			
Background #5	10,797			
Background #6	10,867			
Background #7	10,827			
Background #8	11,130			
Background #9	10,976			
Background #10	11,024	-		
Background #11	10,798			
Background #12	11,019			
Background #13	11,173			
Background #14	11,035			
Background #15	10,740			
Background #16	10,977			
Background #17	10,781			
Background #18	10,941			
Background #19	11,118			

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 869 V, Threshold = 103, Window = 3851 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 10,947 counts Standard Deviation of Measurements: 126 counts

% CV: 1.2 %

Expected Standard Deviation: 105 counts

Table C-1f. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (13 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	10,131	1305	868	5.8
Background #2	10,087			
Background #3	10,087			
Background #4	10,007			
Background #5	10,133			
Background #6	10,266			

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S - 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 868 V, Threshold = 103, Window = 3848 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 10,125 counts Standard Deviation of Measurements: 94 counts

% CV: 0.93 %

Expected Standard Deviation: 101 counts

Table C-1g. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (14 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	11,168	1420	868	5.8
Background #2	11,142			
Background #3	10,861			
Background #4	10,953			
Background #5	11,097			
Background #6	11,139			
Background #7	11,018			
Background #8	10,930			
Background #9	10,915			
Background #10	10,799			
Background #11	10,934			
Background #12	10,957			
Background #13	10,899			
Background #14	10,852			
Background #15	10,832			
Background #16	10,965			
Background #17	10,889			
Background #18	10,645			
Background #19	10,845			
Background #20	10,644			

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 868 V, Threshold = 102, Window = 3845 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 10,923 counts Standard Deviation of Measurements: 145 counts

% CV: 1.3 %

Expected Standard Deviation: 105 counts

Table C-1h. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (14 September 2000).

Location	n Counts (30 s) Time		High Voltage	Battery (V)	
Background #1	10,607	1600 869		5.8	
Background #2	10,525				
Background #3	10,663			-	
Background #4	10,493				
Background #5	10,469				

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S - 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 869 V, Threshold = 102, Window = 3845 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 10,571 counts Standard Deviation of Measurements: 119 counts

% CV: 1.1 %

Expected Standard Deviation: 103 counts

Table C-1i. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (14 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
Background #1	10,340	1620 869		5.8
Background #2	10,275			
Background #3	10,310		,	
Background #4	10,520			
Background #5	10,490			
Background #6	10,231			
Background #7	10,353			
Background #8	10,306			
Background #9	10,235			
Background #10	10,226			
Background #11	10,087			
Background #12	10,150			
Background #13	10,336			
Background #14	10,018	1635	869	5.8

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 863 V, Threshold = 103, Window = 3848 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 10,277 counts Standard Deviation of Measurements: 137 counts

% CV: 1.3 %

Expected Standard Deviation: 101 counts

Table C-1j. Instrument Background and Quality Assurance/Quality Control Measurements [3 x 3 NaI - #1] (15 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)		
Background #1	10,001	1410	869	5.6		
Background #2	10,100					
Background #3	10,123					
Background #4	10,049					
Background #5	10,114					
Former Hot-Spot	10,995					
Former Hot-Spot	11,101	Area Where Formerly the Highest In-Situ				
Former Hot-Spot	11,094	γ-Radiation	n Measurements and So	oil Sampling		
Former Hot-Spot	10,949		Results were Obtained			
Former Hot-Spot	11,036					
Former Hot-Spot	11,145					
Background #1	10,124					
Background #2	10,114					
Background #3	10,096					
Background #4	10,099					
Background #5	10,084	1630	868	5.6		

B1 GPS Location: 40° 39.524 (latitude) 86° 08.181 (longitude) B1 Grid Location: 10S – 0W

Meter: Ludlum Model 2221 Ratemeter/Scaler, Serial Number 97810

Meter Settings: HV = 868 V, Threshold = 103, Window = 3848 (operated in "OUT" position)

Probe: Ludlum 44-20 3 x 3 NaI(Tl), Serial Number 171221 Calibrated: 8 Sep 00

Mean Background Response: 10,077 counts
Standard Deviation of Measurements: 51 counts

% CV: 0.5 % 0.15 %

Expected Standard Deviation: 100 counts
101 counts

Mean Former Hot-Spot Area: 11,053 counts Standard Deviation of Measurements: 73 counts % CV: 0.7 %

Table C-2a. In-Situ Grid Measurements [3 x 3 NaI - #1] (11 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
55N-17W #1	45,481	1330	861	5.9
55N-17W #2	44,874			
55N-17W #3	45,003			
55N-17W #4	44,969			
55N-17W #5	44,647	Mean = 44,995		

Table C-2b. In-Situ Grid Measurements [3 x 3 NaI - #2] (11 September 2000).

Location	Counts (30 s)	(30 s) Time High Vol		Battery (V)
55N-17W #1	50,903	1330 . 740		5.8
55N-17W #2	50,709			
55N-17W #3	50,251			
55N-17W #4	50,316			
55N-17W #5	50,850	Mean = 50,606 (

Table C-2c. In-Situ Grid Measurements [3 x 3 NaI - #1] (11 September 2000).

Location	Counts (30 s)	Time	High Voltage	Battery (V)
50N-10W	10,330	1400	861	5.9
60N-10W	10,734			
50N-20W	10,047		,	
40N-30W	10,413			
23N-52W	10,350			·

Table C-3. Post-Remediation (1st Lift) Grid Measurements (30 s Counts). [3 x 3 NaI - #1]

Location		Location North (meters)							
West (m)	23	36	38	40	42	44	46	48	50
10					11146*	10976**	11478**	11191	10836
12			10920	10841	10910	11100	10765	10802	11141
14					11027	10980	11070	10896	10836
16					10862**	11023	11167	11164	10993
18						10809	10962	11478	11107
20						11264**	11447	11427	11565
22							11337	11380	12640**
28					12265*				
30				11392	11629				
32				11608*					
37		11662						,	
52	11555								

Location	Location North (meters)								
West (m)	52	53	54	55	56	58	60	62	64
6			11166				11331*	11594*	
8			11366		11230**	10946	11063	11131	10995*
10	11150		10909		11073	10849	10722	10998**	
12	10872		11578		11309	11339	10973		
14	11020		12263		11664	10804			
16	11016	11638		12489	11548				, , , , , , , , , , , , , , , , , , , ,
18	11736	11834		11810					
20	11840								
24	12000*								

^{*} Measurement within 30 cm of excavation edge
** Measurement within 100 cm of excavation edge

Table C-4. Post-Remediation (2nd Lift) Grid Measurements (30 s Counts). [3 x 3 NaI - #1]

Grid Locat	Grid Location (meters)		2 nd Lift	Difference	
N-S	E-W	(Counts)	(Counts)	(Counts)	
50N	22W	12,640	11,820	- 820	
52N	20W	11,840	11,680	- 160	
52N	18W	11,736	11,465	- 271	
54N	18W	12,263	11,864	- 399	
54N	16W	11,578	11,499	- 79	
56N	16W	11,548	11,728	180	
54N	14W	10,909	11,085	176	

Table C-5. Post-Remediation (1st Lift) Field Replicate Grid Measurements (30 s Counts) [3 x 3 NaI - #1]

Grid Locat	ion (meters)	1 st	Replicate	Difference
N-S	E-W	Measurement (Counts)		
48N	12W	10,802	11,427	625
50N	14W	10,836	11,600	764
56N	14W	11,664	11,982	318
52N	18W	11,736	11,969	233
48N	18W	11,478	11,714	236
44N	18W	10,809	11,282	473
46N	22W	11,337	11,213	- 124
60N	8W	11,063	11,598	535
40N	30W	11,392	11,340	- 52
36N	37W	11,662	11,451	- 211
			Mean Difference	280

53

Standard Deviation

329

Table C-6. Post-Remediation Soil Sample γ-Spectroscopy Results.

Sample Base Sample		²³⁴ Th (1	pCi g ⁻¹)	²³⁵ U (pCi g ⁻¹)		²⁴¹ Am
Location	Number	Value	Uncertainty	Value	Uncertainty	(pCi g ⁻¹)
19N – 54W	GS0000705	1.3	0.2	< 0.03	NA	< 0.06
26N - 50W	GS0000706	3.6	0.3	< 0.03	NA	< 0.07
26N - 50W	Duke-G56036	3.9	0.7	< 0.04	NA	0.13 ± 0.08
26N – 58W	GS0000707	0.6	0.2	< 0.03	NA	< 0.06
33N – 6W	GS0000708	0.8	0.2	< 0.03	NA	< 0.06
33N – 14W	GS0000709	< 0.51		< 0.03	NA	< 0.06
33N - 14W	Duke-G56037	1.1	0.5	0.08	0.10	0.04 ± 0.07
33N – 30W	GS0000710	73	0.2	< 0.03	NA	< 0.06
33N - 30W	GS0000711	0.8	0.2	0.11	0.01	< 0.06
33N – 38W	GS0000712	1.7	0.2	< 0.03	NA .	< 0.06
33N – 38W	Duke-G56038	1,4	0.7	0.09	0.09	0.16 ± 0.06
40N – 2W	GS0000713	1.2	0.2	< 0.03	NA	< 0.06
40N – 2W	Duke-G56039	1.2	0.6	< 0.03	NA	0.06 ± 0.09
40N – 10W	GS0000714		0.2	< 0.03	NA	< 0.06
40N – 10W	GS0000715	16	0.2	< 0.03	NA	< 0.04
40N – 18W	GS0000716	1.3	0.2	< 0.03	NA	< 0.03
40N – 26W	GS0000717	0.7	0.2	< 0.03	NA	< 0.03
40N – 26W	Duke-G56040	1.1	0.5	< 0.03	NA	< 0.03
40N – 34W	GS0000718	1.7	0.2	< 0.03	NA	< 0.04
40N - 42W.	GS0000719	0.8	0.2	< 0.03	NA	< 0.03
47N – 6W	GS0000720	1.6	0.2	< 0.03	NA	< 0.04
47N – 14W	GS0000721	3.7	0.3	< 0.03	NA	< 0.04
47N – 14W	Duke-G56041	4.3	0.3	0.07	0.09	< 0.02
47N – 22W	GS0000722	1.7	0.2	< 0.03	NA	< 0.04
47N – 30W	GS0000723	1.1	0.2	< 0.03	NA	< 0.03
54N – 2W	GS0000724	1.5	0.2	< 0.03	NA	< 0.04
54N – 2W	Duke-G56042	2.3	0.5	0.12	0.08	0.27 ± 0.11
54N – 10W	GS0000725	0.9	0.2	< 0.03	NA	< 0.04
54N – 18W	GS0000726	7.4	0.5	0.24	0.06	< 0.05
54N – 18W	Duke-G56043	5.9	0.8	0.25	0.10	< 0.07
54N - 26W	GS0000727	0.9	0.2	< 0.03	NA	< 0.04
54N – 26W	Duke-G56044	0.6	0.5	< 0.03	NA	0.03 ± 0.08
61N – 6W	GS0000728	3.5	0.3	< 0.04	NA	< 0.04
61N – 6W	GS0000729	3.44	0.3	< 0.03	NA	< 0.04
61N – 14W	GS0000730	1.0	0.2	< 0.03	NA	< 0.04
61N – 14W	Duke-G56045	1.3	0.7	0.07	0.09	0.05 ± 0.06
61N – 22W	GS0000731	0.9	0.2	< 0.03	NA	< 0.04
68N – 10W	GS0000732	0.7	0.2	0.12	0.09	< 0.04

Uncertainty Values at the 10 Level

Cells Highlighted and Combined Indicate Replicates

NA = Not Applicable

Figure C-1. Post-Remediation In-Situ γ -Radiation Measurements (1st Lift) - (n = 66).

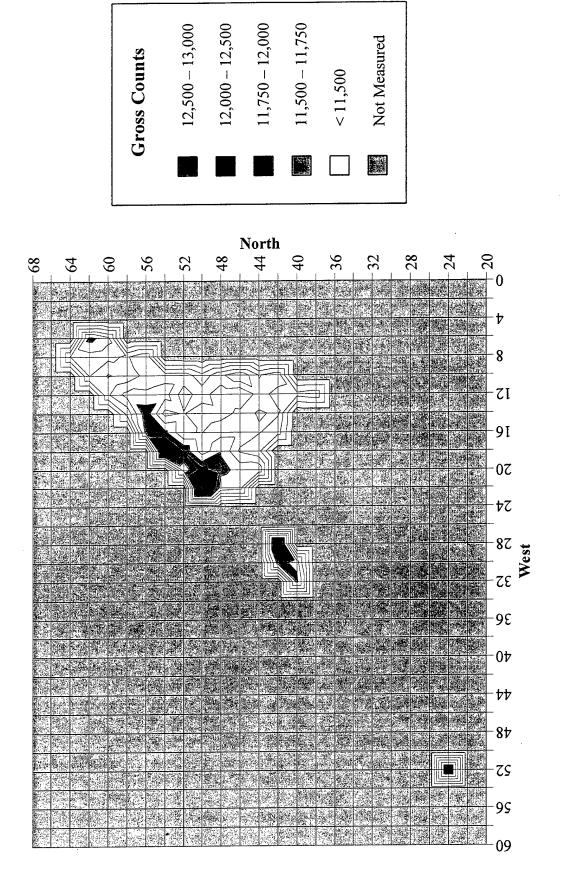
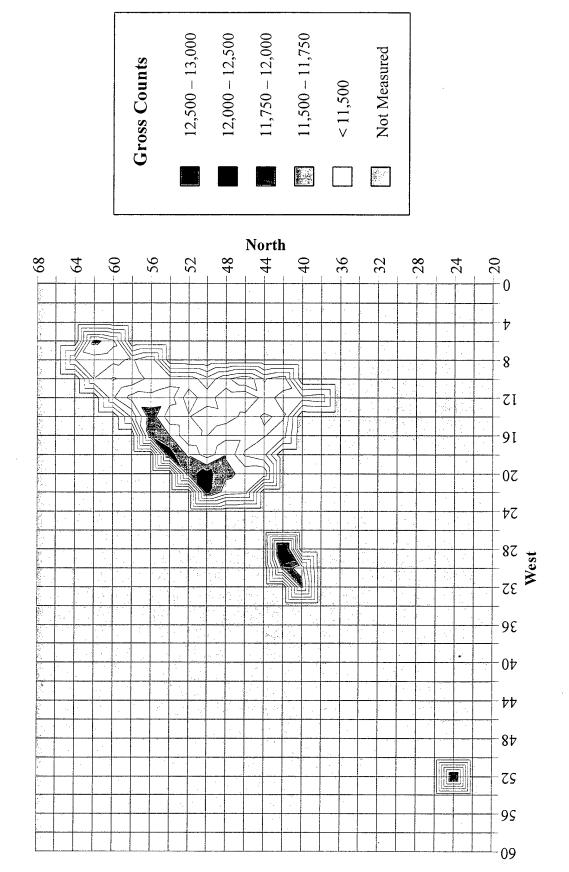


Figure C-2. Post-Remediation In-Situ γ -Radiation Measurements (2nd Lift) - (n = 66).



Appendix D

Complete Private Third-Party (Duke Engineering Services) γ-Spectroscopy Analysis Results

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Analysis Report

Customer: Brooks AFB

Attention: Tsgt. Karen Glaken

Report Date: 10/19/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 28

A26W A54N

Sample Amount: 1.410 Elapsed Time:

ka 19.0417 days .

Lab Sample #: G56044 Sample Code: ZTS 28 3700

Analyses Req: G

Comment: - Activity Decay Conc. $\pm \sigma$ MDC Nuclide Correction /picoCurie/ kg-dry xNp-239 (22 ± (32 ± Co~57 9.52E-01 10)E 00 33E 00 Ce-144 9.54E-01 32 ± 27E 01 80)E 00 Ce-141 6.66E-01 (29 ± 26) E 00 87E 00 xMo-99 Se-75 8.95E-01 (16 ± 16)E 00 53E 00 Cr-51 6.21E-01 (-10 ± 15)E 01 52E 01 I-131 1.93E-01 (-4 ± 53)E 00 11)E 01 19E 01 7.81E-01 Be-7 20 ± 36E 01 Ru-103 7.14E-01 (7 14)E 00 47E 00 xI-133 3.56E-01 Ba-140 (-9 11)E 01 40E 01 (-64 ± (26 ± Cs-134 9.82E-01 89) E-01 42E 00 Ru-106 9.64E-01 98)E 00 34E 01 9.98E-01 529 ± Cs-137 30)E 00 58E 00 (-26 ± Ag-110M 9.48E-01 15)E 00 59E 00 2r-95 8.13E-01 (2 20)E 00 73E 00 (-19 ± Co-58 11)E 00 8.29E-01 43E 00 (902 Mn-54 9.58E-01 12)E 00 50)E 00 42E 00 AcTh228 1.00E 00

1.00E 00 32 ± Notes: Activity greater than 3 standard deviations Peak is found *

1.74E-02

7.43E-01

9.47E-01

9.93E-01

1.00E 00

8.03E-01

1.00E 00

1.00E 00

+

TeI-132

Fe-59

2n-65

Co-60

Sb-124

U-235

Th-234

Ap-241

K-40

Decay correction is less than .01

Reporting level ratio:

0.000

M. Raimondi

15E 01

45E 02

16E 01

20E 01

43E 00

36E 01

68E 00 29E+01

15E+02

27E+01

Approved by

(-10

(-19

(-45

1363

(3 ±

-37 ±

5 ±

57 ±

±

±

13)E 02

42)E 00

57) E 00

12)E 00

49)E 01

17)E 00

83)E 00 46) E+01

78)E+00

Analysis Report

Brooks AFB Customer:

Attention: Tsgt. Karen Glacken

Report Date: 10/19/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 29

A14W A61N

Sample Amount: 0.504 Elapsed Time:

Comment:

kq

19.0442 days

Lab Sample #: Sample Code: 2TS 29 3700

Analyses Req: G

Activity Decay Conc. $\pm \sigma$ MDC Nuclide Correction [picoCurie/ kg-dry] xNp-239 Co-57 9.52E-01 $(12 \pm$ 11)E 00 38E 00 Ce-144 9.54E-01 (-1 # 92)E 00 32E 01 Ce-141 6.66E-01 15 ± 29)E 00 98E 00 xMo-99 Se-75 8.95E-01 21 ± 17)E 00 58E 00 Cr-51 6.20E-01 21 ± 17)E 01 (21 ± (73 ± 56E 01 I-131 1.93E-01 67)E 00 22E 01 (8 Be-7 7.81E-01 15)E 01 ± 52E 01 7.14E-01 Ru-103 (-10 17)E 00 62E 00 xI-133 Ba-140 3.56E-01 (8 41E 01 12)E 01 `28 (~4 19)E 00 13)E 01 Cs-134 9.82E-01 ± 64E 00 Ru-106 9.64E-01 ± 47E 01 Cs-137 9.98E-01 478 ± 35) E 00 73E 00 Aq-110M 9.48E-01 (2 ± 19)E 00 67E 00 Zr-95 8.13E-01 49 27)E 00 # 89E 00 Co-58 8.29E-01 (-19 ± 14)E 00 56E 00 Mn-54 12) E 00 55) E 00 9.58E-01 (5 ± 44B 00 AcTh228 1.00E 00 (691 ± 19E 01 TeI-132 1.73E-02 (-19)10)E 02 ± 40E 02 Pe-59 7.43E-01 -17 56) E 00 21E 01 30 Zn-65 9.47E-01 ± 68)E 00 23E 01 Co-60 9.93E-01 -47 ± 15)E 00 66E 00 1369 ± 57)E 01 K-40 9.99E-01 43E 01 Sb-124 ± 11E 01 8.02E-01 70 29)E 00 **U-235** 1.00E+00 91)E 00 31E+01 Th-234 1.00E+00 133 ± 67) E+01 22E+02 1.00E+00 Am-241 58)E 00 54 19E+01 Notes: Approved by

* Activity greater than 3 standard deviations

Peak is found

Decay correction is less than .01

Reporting level ratio:

0.000

10/19/00 Raimondi

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Analysis Report

Customer: Brooks AFB

Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 20

A50W A26N

Sample Amount: 1.359 Elapsed Time:

Comment:

18.9767 days

Lab Sample #: G56036 Sample Code: ZTS 20 3700

Analyses Req: G

Commette.		Activity —			
Nuclide	Decay Correction	Conc. ± σ [picoCurie/]	MDC		
xNp-239					
Co-57	9.52E-01	(1 ± 12)E 00	42E 00		
Ce-144	9.54E-01	$(-52 \pm 94) \to 00$	33E 01		
Ce-141	6.67E-01	(40 ± 33)E 00	11E 01		
xMo-99		***************************************			
Se-75	8.95E-01	(9 ± 20)E 00	69E 00		
Cr-51	6.22E-01	$(4 \pm 18)E01$	61E 01		
I-131	1.94E-01	(-12 ± 68) E 00	24E 01		
Be-7	7.81E-01	$(13 \pm 14) E 01$	47E 01		
Ru-103	7.15E-01	(0 ± 18)E 00	63E 00		
xI-133					
Ba-140	3.57E-01	$(3 \pm 14) E 01$	49E 01		
Cs-134	9.82E-01	(-10 ± 13) E 00	50E 00		
Ru-106	9.64B-01	$(2 \pm 15) = 01$	52E 01		
+* Cs-137	9.98E-01	(378 ± 33) E 00	72E 00		
Ag-110M	9.48E-01	(0 ± 18)E 00	68E 00 11E 01		
Zr-95	8.14E-01	(-28 ± 29)E 00			
Co-58	8.30E-01	$(-12 \pm 12) E 00$	50E 00-		
Mn-54	9.58E-01	(-15 ± 14) E 00	53E 00 22E 01		
+* AcTh228	1.00E 00	(779 ± 68) E 00 (-21 ± 18) E 02	65E 02		
TeI-132	1.76E-02	\ -	22E 01		
Fe-59	7.44E-01	(3 ± 59)E 00 (126 ± 69)E 00	23E 01		
Zn-65	9.47E-01	(126 ± 69)E 00 (-10 ± 14)E 00	56E 00		
Co-60	9.93E-01	(1342 ± 63)E 01	63E 01		
+* K-40	9.99E-01	(+33 ± 24) E 00	12E 01		
Sb-124	8.03E-01	(-10 ± 10)E+01	36E+01		
U-235	1.00E 00	· · · · · · · · · · · · · · · · · · ·	25E+02		
· . IU-234 .	1.00E 00	(385 ± 72)E+01	25E+01		
Am-241 Notes:	1.00E 00	(133 ± 75)E 00	Approved by		

* Activity greater than 3 standard deviations + Peak is found x Decay correction is less than .01

Reporting level ratio:

0.000

Raimondi

<u>Analysis Report</u>

Brooks AFB Customer:

Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 21

A14W A33N

Sample Amount: 1.366 Elapsed Time:

kg 18.9778 days

Lab Sample #: G56037 Sample Code: ZTS 21 3700

Analyses Req: G

Comment: - Activity Decay Conc. $\pm \sigma$ MDC Nuclide Correction spicoCurie/ kg-dry 1 xNp-239 Co-57 9.52E-01 (-6 ± 12)E 00 42E 00 9.54E-01 20 ± Ce-144 96)E 00 33E 01 6.67E-01 70 Ce-141 33)E 00 11E 01 xMo-99 (3 (25 Se-75 8.95E-01 68E 00 ± 20)E 00 Cr-51 6.21E-01 ± 19)E 01 62E 01 (-70 ± 1.94E-01 72)E 00 17)E 01 I-131 26E 01 Be-7 7.81E-01 23 ± 58E 01 63E 00 7.15E-01 16 18)E 00 Ru-103 xI-133 3.57E-01 53E 01 Ba-140 (-11 ± 14)E 01 (-10 14)E 00 Cs-134 9.82E-01 51E 00 (-15 ± 9.64E-01 Ru-106 12)E 01 49E 01 9.98E-01 (458 ± 73E 00 Cs-137 36)E 00 77E 00 9.48E-01 ± Ag-110M (13 22)E 00 ZÍ-95 ± 8.14E-01 (-40 30)E 00 12E 01 (0 16)E 00 Co-58 8.30E-01 ± 58E 00 2 Mn-54 9.58E-01 16)E 00 60E 00 ± 1.00E 00 (857 23E 01 43E 02 AcTh228 72)E 00 TeI-132 1.76E-02 (-11 11)E 02 Fe-59 7.44E-01 -81 ± 60)E 00 24E 01 9.47E-01 (117 87)E 00 29E 01 Zn-65 Co-60 9.93E-01 18)E 00 61E 00 (21 ± 69)E 01 19)E 00 K-40 9.99E-01 1371 ± 79E 01 90E 00 ± Sb-124 **(** · 0 8.03E-01 33E+01 78 ± 99)E 00 **U-235** 1.00E 00 17E+02 1.00E 00 109 ± 53) E+01 Th-234 22E+01 ± Am-241 1.00E 00 43 66)E 00 Notest Approved by

Activity greater than 3 standard deviations

Peak is found

Decay correction is less than .01

Reporting level ratio:

0.000

10/18/00 Raimondi

Analysis Report

Customer: Brooks AFB Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 22 ASSW ASSN

Sample Amount: 1.458 Rlapsed Time:

Lab Sample #: G56038 Sample Code: ZTS 22 3700

PTC	Comment:	10.3/31 days		Analyses Re	eq: G
	.v	Decay	Conc	Activ t σ [picoCurie/	MDC
***********	Nuclide	Correction		[DICOCULTE/	ing one !
	xNp-239		-		
	Co-57	9.52E-01	(− 18 ±	11)E 00	40E 00
	Ce-144	9.54E-01	(-100 ±	92)E 00	32E 01
	Ce-141	6.67E-01	`(36 ±	31)E 00	10E 01
	xMo-99		` -		
	Se-75	8.95E-01	(−45 ±	20)E 00	75E 00
	Cr-51	6.21E-01	(− 25 ±	18)E 01	66E 01
	I-131	1.94E-01	(48 ±	65)E 00	22E 01
	Be-7	7.81E-01	(12 ±	14)E 01	49E 01
,	Ru-103	7.15E-01	(12 ±	17)E 00	59E 00
	xI-133		· ÷		
	Ba-140	3.57É-01	(13 ±	14)E 01	48E 01
	Cs-134	9.82E-01	(-5 ±	12)E 00	48E 00
	Ru-106	9.64E-01	(- 7 ±	14)E 01	52E 01
+*	Cs-137	9.98E-01	· (417 ±	33)E 00	64E 00
	Ag-110M	9.48E-01	(-38 ±	23)E 00	90E 00
	Zr-95	8.14E-01	(31 ±	27) E 00	92E 00
	Co-58	8.30E-01	(-15 ±	14)E 00	56E 00
	Mn-54	9.58E-01	`(2 ±	17)E 00	62 E 00
+*	AcTh228	1.00E 00	(786 ±	65)E 00	20E 01
	TeI-132	1.762-02	(20 ±	19)E 02	64E 02
	Fe-59	7.44E-01	(19 ±	54)E 00	20E 01
	Zn-65	9.47E-01	(-114 ±	76)E 00	28E 01
	Co-60	9.93E-01	(-29 ±	16)E 00	69E 00
+*	K-40	9.99E-01	(i486 ±	68)E 01	66E 01
	Sb-124	8.03E-01	(27 ±	20)E 00	66E 00
	U−23 5	1.00E 00	(92 ±	94)E 00	36E+01
+	Th-234	. 1.00E 00	(141 ±	73) E+01	24E+02
	Am-241	1.00E 00	(162 ±	62)E 00	20E+01
Note	s:			•	Approved by

* Activity greater than 3 standard deviations + Peak is found

x Decay correction is less than .01

Reporting level ratio:

0.000

Analysis Report

Customer: Brooks AFB Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 23

A02W A40N

Sample Amount: 1.382 Elapsed Time:

Comment:

18.9803 days

Lab Sample #: G56039
Sample Code: ZTS 23 3700 Analyses Req: G

			Activity	
		Decay <u>Correction</u>	Conc. $\pm \sigma$	MDC J-dry/j
	Nuclide	COFFECTION		
				
	xNp-239	0 507 01	(11 ± 11)E 00	38E 00
	Co-57	9.52E-01	(-78 ± 87)E 00	31E 01
	Ce-144	9.54E-01	(13 ± 28)E 00	95E 00
	Ce-141	6.67E-01	(13 1 20/1 00	
	xMo-99		(-23 ± 18)E 00	64E 00
	Se-75	8.95E-01		58E 01
	Cr-51	6.21E-01		22E 01
	I-131	1.94E-01		41E 01
	Be-7	7.81E-01		50E 00
	Ru-103	7.15E-01	(14 ± 15)E 00	50E 00
	xI-133			51E 01
	Ba-140	3.57E-01	$(-25 \pm 13) \times 01$	40E 00
	Cs-134	9.82E-01	$(-6 \pm 11) E 00$	44E 01
	Ru-106	9.64E-01	$(-17 \pm 11) E 01$	50E 00
+*	Cs-137	9.98E-01	(450 ± 29)E 00	
	Ag-110M	9.48E-01	$(9 \pm 18) E 00$	62E 00 81E 00
	2r-95	8.14E-01	$(21 \pm 24) E 00$	81E 00 52E 00
	Co-58	8.30E-01	$(-16 \pm 13) E 00$	
	Mn-54	9.58E-01	(-5 ± 14)E 00	50E 00
+*	AcTh228	1.00E 00	(728 ± 53)E 00	18E 01 37E 02
	TeI-132	1.76E-02	(-235 ± 91)E 01	
	Fe~59	7.44E-01	$(-13 \pm 47) E 00$	17E 01
	Zn-65	9.47E-01	$(-100 \pm 63) E 00$	23E 01
	Co-60	9.93E-01	$(11 \pm 12) E 00$	42E 00
+*	K-40	1.00B 00	(1419 ± 55)E 01	44E 01
-	Sb-124	8.03E-01	$(12 \pm 15) E 00$	58E 00 30E+01
	บี-235	1.00E+00	(`-97 ± 86)E 00	• •
+	Th-234	1.00E+00	(124 ± 58)E+01	25E+02
•	Am-241	1.00E+00	(63 ± 85)E+00	29E+01
Note		1.002.00	•	Approved by
-104			ndard deviations	

* Activity greater than 3 standard deviations + Peak is found x Decay correction is less than .01

Reporting level ratio:

0.000

Analysis Report

Customer: Brooks AFB

Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00

Reference Date: 09/14/00

Soil

Station No: 24 A26W A40N

Sample Amount: 1.459 ka .

18.9812 days Elapsed Time: Comment:

Lab Sample #: G56040 Sample Code: 2TS 24 3700

Analyses Req: G

· Activity -Conc. $\pm \sigma$ MDC Decay [picoCurie/, kg-dry Nuclide Correction xNp-239 Co-57 9.52E-01 10)E 00 (-12 ± 36E 00 (~16 ± (36 ± 82)E 00 26)E 00 Ce-144 9.54E-01 28E 01 85E 00 Ce-141 6.67E-01 xMo-99 (-7 ± Se-75 8.95E-01 17)E 00 61E 00 (-15 ± 15)E 01 6.21E-01 Cr-51 54E 01 (-4 ± (9 ± I-131 1.94E-01 56)E 00 20E 01 12)E 01 14)E 00 7.81E-01 7.15E-01 42E 01 Be-7 (-11 ± 52E 00 Ru-103 xI-133 (8 ± 3.57E-01 12)E 01 40E 01 Ba-140 (0 ± (-5 ± 12)E 00 41E 00 Cs-134 9.82E-01 39E 01 Ru-106 9.64E-01 11)E 01 (628 ± Cs-137 9.98E-01 32)E 00 51E 00 15)E 00 Ag-110M Zr-95 (-6 ± 9.48E-01 57E 00 8.14E-01 (18 ± 20)E 00 67E 00 (30 ± 19)E 00 8.30E-01 63E 00 Co-58 12)E 00 55)E 00 (15 ± 826 ± 41E 00 16E 01 9.58E-01 Mn-54 AcTh228 1.00E 00 (21 ± TeI-132 1.76E-02 13)E 02 43E 02 38)E 00 Fe-59 7.44E-01 14E 01 (-113 ± 9.47E-01 64)E 00 23E 01 Zn-65 11)E 00 52)E 01 9.93E-01 Co-60 (14 ± 38E 00 1448 ± 37E 01 9.98E-01 K-40 $(14 \pm$ 68E 00 Sb-124 8.03E-01 18)E 00 27E+01 U-235 1.00E 00 2 ± 78)E 00 1.00E 00 17E+02 Th-234 108 ± 51) E+01 28E+01 Am-241 1.00E 00 79)E 00 Notes: Approved by

Activity greater than 3 standard deviations Peak is found

Decay correction is less than .01

Reporting level ratio:

0.000

Mainundi 10/18/00 M. Raimondi

C

Analysis Report

Customer: Brooks AFB

Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 25

A14W A47N

Sample Amount: 1.601 Elapsed Time: Comment:

18.9821 days

Lab Sample #: G56041 Sample Code: 2TS 25 3700 Analyses Req: G

				maryses r	vity —
		Decay	Conc	± σ	
	Nuclide	Correction	COLIC	[picoCurie	kg-dry MDC
	xNp-239	•			-
	Co-57	9.52E-01	(−10 ±	10)E 00	37E 00
	Ce-144	9.54E-01	(-101 ±	87) E 00	30E 01
	Ce-141	6.67E-01	`(24 ±	28)E 00	92E 00
	xMo-99		,		
	Se-75	8.95E-01	(10 ±	14)E 00	49E 00
	Cr-51	6.21E-01	(-14 ±	15)E 01	54E 01
	I-131	1.94E-01	(`105 ±	50)E 00	16E 01
	Be-7	7.81E-01	(-5 ±	11)E 01	40E 01
	Ru-103	7.15E-01	(13 ±	12)E 00	42E 00
	xI-133		,		
	Ba-140	3.57E-01	(−43 ±	92)E 00	34E 01
	Cs-134	9.82E-01	`(- 8 ±	11)E 00	40E 00
	Ru-106	9.64E-01	(6 ±	10)E 01	35E 01
+*	Cs-137	9.98E-01	(`93 ±	20)E 00	57E 00
	Ag-110M	9.48E-01	`(~8 ±	14)E 00	52E 00
	Zr-95	8.14E-01	(42 ±	51)E 00	17E 01
	Co-58	8.30E-01	(− 25 ±	11)E 00	46E 00
_	Mn-54	9.58E-01	(8 ±	12)E 00	40E 00
+*	AcTh228	1.00E 00	(770 ±	50)E 00	17E 01
	TeI-132	1.76E-02	(−69 ±	88)E 01	33E 02
	Fe-59	7.44E-01	(53 ±	43)E 00	15E 01
	2n-65	9.47E-01	(30 ±	59)E 00	20E 01
	Co-60	9.93E-01	(-8 ±	11)E 00	44E 00
+*	K-40	9.98E-01	(1335 ±	51)E 01	38E 01
	Sb-124	8.03E-01	(0 ±	22)E 00	89E 00
_	U-235	1.00E 00	(66 ±	87)E 00	29E+01
+*	Th-234	1.00E 00	(<u>4</u> 27 ±	31) E+01	92E+01
Note	Am-241	1.00E 00	(-6 ±	66) E 00	23E+01
NOCE		greater than 3 standard	danda+1	-	Approved by

* Activity greater than 3 standard deviations + Peak is found x Decay correction is less than .01

Reporting level ratio:

0.000

10/18/00

<u>Analysis Report</u>

Customer: Brooks AFB Attention: TSgt. Karen Glaken

Report Date: 10/09/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 26

A02W A54N

ample Amount: 1 1.310
\$lapsed Time: 19.0

Comment:

19.0409 days

Lab Sample #: G56042 Sample Code: ZTS 26 3700 Analyses Req: G

Commente.	•		Activity	
	Decay	Conc.	. ± σ	MDC
Nuclide	Correction		[picoCurie/ kg-d	ry
-Mt- 220				
xNp-239	2 525 21	, <u> </u>	4.4.1720.0	255
Co-57	9.52E-01	{-3 ±	10)E 00	36E 00
Ce-144	9.54E-01	(-174 ±	82)E 00	30E 01
Ce-141	6.66E-01	(48 ±	29)E 00	94E 00
xMo-99				
8e-75	8.95E-01	(5 ±	18)E 00	62E 00
Cr-51	6.20E-01	(17 ±	16)E 01	55E 01
I-131	1.93E-01	(-119 ±	61)E 00	23E 01
8e-7	7.81E-01	(7 ±	13)E 01	47E 01
ų=103	7.14E-01	(7 ± '	16)E 00	56E 00
. -133		-		
Ba-140	3.56E-01	(6±	13)E 01	47E 01
Cs-134	9.82E-01	(10 ±	34)E 00	12E 01
Ru-106	9.64E-01	(−5 ±	11)E 01	42E 01
Cs-137	9.98E-01	(323 ±	30)E 00	69E 00
Ag-110H	9.48E-01	(-4 ±	15}B 00	58E 00
Zr-95	8.13E-01	(12 ±	24)E 00	84E 00
Co-58	8.29E-01	(-10 ±	13)E 00	51E 00
Mn-54	9.58E-01	(−16 ±	14)E 00	53E 00
AcTh228	1.00E 00	(530 ±	57)E 00	23 E 01
TeI-132	1.74E-02	`(5 ±	17)E 02	57E 02
Fe- 59	7.43E-01	(SO ±	53)E 00	18E 01
Zn-65	9.47E-01	(-61 ±	74)E 00	26E 01
Co-60	9.93E-01	(15 ±	13)E 00	43E 00
: K-40	9.97E-01	(Ì420 ±	59)E 01	54E 01
Sb-124	8.03E-01	(O ±	24) E 00	98E 00
U-235	1.00E 00	(122 ±	88)E 00	29E+01
Th-234	1.00E 00	(228 ±	49) E+01	16E+02
Am-241	1.00E 00	(27 ±	11)E+01	34E+01
tes:	2002 00	, , , ,	,-·	ipproved by

Activity greater than 3 standard deviations Peak is found Decay correction is less than .01

porting level ratio:

0.000

Analysis Report

Customer: Brooks AFB

Attention: TSgt. Karen Glacken

Report Date: 10/19/00 Analysis Date: 10/03/00 Receipt Date: 09/19/00 Reference Date: 09/14/00

Soil

Station No: 27 A18W A54N

Sample Amount: 0.568 Elapsed Time:

19.0431 days

Lab Sample #: G56043 Sample Code: ZTS 27 3700 Analyses Req: G G56043

Comment:

			Additivity —	
	Nuclide	Decay Correction	Conc. ± o [picoCurie/kg-dry	MDC
				
	хµр-539	· · · · · · · · · · · · · · · · · · ·		
	Co-57	9.52E-01	(-4 ± 13)E,00	44E 00
	Ce-144	9.54E-01	(−5 ± 97)E 00	33E 01
	Ce-141	6.66E-01	(26 ± 32)E 00	11E 01
	xMo-99			
	Se-75	8.95E-01	(-22 ± 19)E 00	67E 00
	Cr-51	6.20E-01	$(2 \pm 17) E 01$	58E 01
	I-131	1.93E-01	(-16 ± 68)E 00	24E 01
	Be-7	7.81E-01	(9 ± 13)E 01	45E 01
	Ru-103	7.14E-01	(24 ± 17)E 00	56E 00
	xI-133	 .		
	Ba-140	3.56E-01	(-10 ± 12)E 01	46E 01
	Cs-134	9.82 E-0 1	(27 ± 22)E 00	73E 00
	Ru-106	9.64E-01	(6 ± 12)E 01	41E 01
	Cs-137	9.98E-01	(38 ± 15)E 00	48E 00
	Ag-110M	9.48E-01	· · · · · · · · · · · · · · · · · · ·	58E 00
	Zr-95	8.13E-01		97E 00
	Co-58	8.29E-01		63E 00
	Mn-54	9.58E-01		52E 00
+*	AcTh228	1.00E 00	$(736 \pm 64) = 00$	20E 01
	TeI-132	1.73E-02	· · · · · · · · · · · · · · · · · · ·	61E 02
	Fe-59	7.43E-01		16E 01
	Zn-65	9.47E-01		23E 01
	Co-60	9.93E-01		52E 00
+*	K-40	9.99E-01		60E 01
	Sb-124	8.02E-01		13E 01
	U-235	1.00E+00		32E+01
+*	Th-234	1.00E+00		33E+02
	Am-241	1.00E+00	: 121 T 121 T	74E+01
Note	es:			oved by

Activity greater than 3 standard deviations Peak is found

x Decay correction is less than .01

Reporting level ratio:

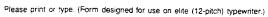
Appendix E

Waste Manifest

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P.O. Box 13087

Austin, Texas 78711-3087





UNIFORM HAZARDOUS 1. Generator's US EPA ID No Manifest	Form approved. OMB No. 2050-0039.
WASTE MANIFEST IN 9 540024472 Comment No.	2. Page 1 Information in the shaded areas is not required by Federal law.
3. Generator's Name and Mailing Address Sufficer Champy FOR	
Anso-Act-D Kock Island, IL 61299 Geisson Alette	
4. Generator's Phone (309) 182-0338 GLISSON ARB, IN	
5. Transporter 1 Company Name 6. US EPA ID Number	
WILLS TRUCKING	
9 Designated Facility Name and Site Address 10. US EPA ID Number WASTE CONTROL SPECIALISTS	
11210 W. BROADWAY	
ANDROUS, TX 61299 TXD988088464	
11. US DOT Description (including Proper Shipping Name, Hazard Class, ID Number and Packing Group) 12. Contain No.	ers 13. 14. Unit Type Quantity Wt/Vol
AWASTE, NON RCRA, NON REGULATED	
SOIL AND GRASS DOT RADIDACTIVE CLEMPT DOIL	2M239051P 225341
N B B B B B B B B B B B B B B B B B B B	
A T O	
Ř C.	
d.	
·	
15. Special Handling Instructions and Additional Information, WASTE PROFILE - WP CONTAINER ID - BKRU 25569	/35/8
NOLUME OF WASTE- 10.0 4d ³	Defleted ulaplum - 0.1% ma
16. GENERATOR'S CERTIFICATION: I hereby declare that the contents of this consignment are fully and accurately classified, packaged, marked, and labelled/placarded, and are in all respects in proper condition for transport by national government regulations, including applicable state regulations.	described above by proper shipping name and are highway according to applicable international and
If I am a large quantity generator, I certify that I have a program in place to reduce the volume and toxicity of was	te generated to the degree I have determined to be
economically practicable and that I have selected the practicable method of treatment, storage, or disposal current future threat to human health and the environment; OR, if I am a small quantity generator, I have made a good fait the best waster management method that is quickled to	h effort to minimize my waste generation and select
the best waste management method that is available to me and that I can afford. Signature Signature	Month Day Year
KUHAD THATCHER U.S ARMY RUGER D	That 10 1/8/00
17. Transporter 1 Acknowledgement of Receipt of Materials Printed/Typed Name Signature	Date
Printed/Typed Name Signature Signature	Month Day Year
18. Transporter 2 Acknowledgement of Receipt of Materials Printed/Typed Name Signature	Date
T Printed/Typed Name Signature	Month Day Year
19. Discrepancy Indication Space	
Page	
20. Facility Owner or Operator: Certification of receipt 71	cept as noted in Item 19.
Printed/Typed Name	Month Day Year

P.O. Box 13087

Austin, Texas 78711-3087



Please print or type. (Form designed for use on elite (12-pitch) typewriter.)

4	UNIFORM HAZARDOUS 1. Generator's US E WASTE MANIFEST LN 9.54.0	PAID NO 24472 PO	Manifest 2. P	age 1	Informati	on in the shaded areas
	3. Generator's Name and Mailing Address US HEIN() OFERA 11005 SUFFORT Common D	150	WIO !	11111	is not re	quired by Federal law.
	Ams 10 ACE-D ROCK ISLAND, IL 6/299	Gerena Ale	Royal			
	4. Generator's Phone (309) 262-0338	GEISSON AR	R.IN			COUPLE
	5. Transporter 1 Company Name 6. WILLS TRUCKING OF	US EPA ID Number	1409	le or		
	7. Transporter 2 Company Name 8.	US EPA ID Number				
	Designated Facility Name and Site Address	US EPA ID Number				
	Waste Conteol Specialists 1710 W. Broadway	OO EI AID Number				
	ANDREWS, TX 61299 IT	×D988088	464	D.		
	11A. 11. US DOT Description (including Proper Shipping Name, H Number and Packing Group)	lazard Class, ID	12. Containers	13 Tot		4.
	& WASTE, NON RORA, NON RE	GULATED	No. Type	Quar		/Vol xonilina
GII	SOIL AND GRASS (DOT RADIOAC	וש משוא שוח	ØØ1cm	187	90 F	
Guzucar	b.	ite chemit	, , , ,	. 0 /	7 - 1	
0						
R	c.					
	d.					
100	Addition a loss princips of barriers of the latest and the latest			310 (50		es cale disease
200						
	15. Special Handling Instructions and Additional Information WA	ISTE YROFILE-	ωP1357	A		
		,,	_	_		
$ \cdot $	16. GENERATOR'S CERTIFICATION: I hereby declare that the contents of classified, packaged, marked, and labelled/placarded, and are in all reviews.	this consignment are fully ar	and an arranged at			1-0.138 mai
	national government regulations, including applicable state regulations	specia in proper condition for	transport by night	way accord	ling to applic	able international and
	If I am a large quantity generator, I certify that I have a program in place economically practicable and that I have selected the practicable method future threat to human health and the environment OB if I am a small o					
	the best waste management method that is available to me and that I car		le a good faith effor	rt to minimi	ze pry waste	generation and select
	Stinted/Typed Name ACITER ACEST OF	Signature K	sel XX	1/7		Month Day Year
	7. Transporter 1 Acknowledgement of Receipt of Materials Printed/Typed Name					Date
	GELOGE TACKSON	Signature	oran lat		n	Month Day Year
1	Transporter 2 Acknowledgement of Receipt of Materials Printed/Typed Name	IS:				Date
	•	Signature				Month Day Year
1	9. Discrepancy Indication Space					
1		Page				
2	D. Facility Owner or Operator: Certification of receipt of h	_	est except a	s noted in	Item 19.	
\vdash	Printed/Typed Name	72	***************************************			Date Month Day Year
I						Month Day Year

Austin. Texas 78711-3087





PI	ease p	orint or type. (Form designed for use on elite (12					orm approved. UNIE	No. 20:	50-0039.
		UNIFORM HAZARDOUS WASTE MANIFEST	1. Generator's US EPA	244726			is not	requir	n the shaded areas ed by Federal law.
	3. Č.	Generator's Name and Mailing Address ABRERA SERVICES FOR S. ALMY, CPERATIONS		AF KOO			He Maritest Dbc	iment	Number 152
		SALMY OPERATIONS TO SIGN SOLVERS	SUFFICET COMMAN		KESERVE	9.9	nie Sarierator's II) 3.E	
	1/ 1/	Generator's Phone (309) 782	08860,0338	GASSOM, A				999	716
	1 -	Transporter 1 Company Name	io 47	US EPA ID Numbe			ne Transporteris		Fig. 1
	منسبا	<u>ÜLLS TEAUS FORT</u> Transporter 2 Company Name	<u> </u>	US EPA ID Numbe			nieperterszenem Majzansponter's		3345 8(22.1) 6.5
		,					nspoders Phon		N. L. S. L. S.
	9.	Designated Facility Name and Site Addre	ss 1575 LLC 10.	US EPA iD Numbe	r	G SI	de Facility's IDS	5 9	
	; ,	U W. BEONDWAY ,		> 0 6 6 6 6 6 7	Arx	H.Fa	ility's Phone		
	1.6	WDEEUS, TX 79714		D988088	· · · · · · · · · · · · · · · · · · ·		100 1207	14.	<i>ซ</i> る:
	11/ HA	Number and Packing Group)			12. Contain No.	ners Type	Total Quantity	Unit Wt/Vol	Waste No.
		" WASTE, NON RCR	A, NON KEGU	LATED .			11810	0	
GEN		SOIL AND GRASS	(DOT RADIOAC	TIVE ELEMPI	991	CM	26020	T	Outs 3191
7 8 8 7		b.		,].				
OT	<u></u>		· · · · · · · · · · · · · · · · · · ·			·			
		c.						i	
		d.							
	J. A	dditional Descriptions for Materials Listed	l'Above :		ing property and the		rdling Codes for \	Vastes	Listed Above
				्राप्त के किया है। अस्ति के किया	39(7.593). 7 477				
	15.	Special Handling Instructions and Additio	nal Information WAST	E PROFILE-	WP1.	35/	8		
	1	OLLIME OF WASTE-	8.3013	DER	ETED	URY	NIUM - C	14	6 m C;
	116	GENERATOR'S CERTIFICATION: Liberary d	eclare thất the contents of thi	s consignment are fully	and accurate	lv desc	ribed above by pro	er ship	ping name and are
	1	classified, packaged, marked, and labelled/pl national government regulations, including app	olicable state regulations.						
	1	If I am a large quantity generator, I certify that economically practicable and that I have selec	ted the practicable method of	treatment, storage, or d	isposal curre	ntly ava	ulable to me which	minimiz	es the present and
П		future threat to human health and the environi the best waste management method that is av			ade a good fa			aste ge	neration and select
$ \downarrow$	H	Printed/Typed Name	ACOUTOF	Signature	1/4		#		Month Day Year
1	<u> </u>	KICHAED THATCHER	U.S. Femi	140	100 C	<u> </u>	il.		7.0 1.9 0.C
A		Transporter 1 Acknowledgement of Rece Printed/Typed Name		Signature					Month Day Year
4200	11	im wolff		Morn					1011800
! 0		Transporter 2 Acknowledgement of Rece		77				l	Date Month Day Yea
שני- ושמי		Printed/Typed Name .		Signature				ļ	l l
۲	19.0	Discrepancy Indication Space						·	
F									
FAC			Page						
	20.	acility Owner or Operator: Certification of	of receipt		ianifest e	xcept a	s noted in Item 1	9.	5
1		Drinted/Transl Nome	73						Date Month Day Yea
1		Printed/Typed Name							

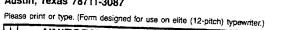
TNPCC-0311 (Rev 07/01/97)

flow-Transporter Green-Generator's first copy

UNIFORM HAZARDOUS

P.O. Box 13087

Austin, Texas 78711-3087

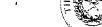




UNIFORM HAZARDOUS 1. Generator's US	EPA ID No	4	Form approved. OMB	No. 2050-0039.
WASTE MANIFEST IN 9540	D 24472	vianifest 2. F	Page 1 Informa	ation in the shaded area
3. Generator's Name and Mailing Address U.S., ARMY OPERATIONS Sulfort Commit Amsio-Ace-D Rock Island, IL 612		100 X	IS NOT	required by Federal la-
Ansia Are a Poor To and Ti	pod for			
A Community 1200, 200, 200, 200, 200, 200, 200, 200		Lesse		
4. Generator's Phone (309) 282-0338 5. Transporter 1 Company Name	GRISSOM, A	28. IN		
WILLS TRUCKING 10	HD 06891	1-0		
7. Transporter 2 Company Name	<u> </u>	M OY M		
8.	US EPA ID Number			
9. Designated Facility Name and Site Address 10.	US EPA ID Number			
WASTE CONTROL SPECIALISTS, LLC	US EFAID NUMBER			
	_			
ANDREWS, TX 79714	<u> 88088 PCX</u>	464		c breeze
11A. 11. US DOT Description (including Proper Shipping Name, H Number and Packing Group)	lazard Class, ID	12. Containers	13.	14
		No. Type	Total Quantity W	Unit /t/Vol
& WASTE, NON ROPA, NON REC				
SOIL AND GRASS (DOT RADION	ICTIVE Exempt)	ØØ.I Cm	19190	PMEST
b.	Janes Sierrig D			
A T				
O C.				
d.				The second secon
- NOTA MELLON CO.		[.]		
				7.16-2-1
15. Special Handling Instructions and Additional Information	STELDATILE			
CONTAINER ID - BKRU 12593	ISTE PROFILE-	WY135	18	
VOLUME OF WASTE- 8 cd3	2000		14.5	- 10
16. GENERATOR'S CERTIFICATION: I boroby dealer	this considerate of the	e led uk	ANIUM - (3.14 mls
"" " " " " " " " " " " " " " " " " " "	30, 0. 0.0	osal currently avai	arated to the degree I lable to me which min	have determined to be imizes the present and
the best waste management method that is available to me and that I can	afford.	a good faith effort	to minimize my waste	generation and select
Printed Typed Name AGENT OF	Signature	} { } +/**	//	Month Day V
MUNREY ITHIUTE IS JOS U	Klick	XXX	•	Month Day Year
17. Transporter 1 Acknowledgement of Receipt of Materials Printed/Typed Name				Date
RILL CRANE	Signature /	-1	1	Month Day Year
18. Transporter 2 Acknowledgement of Receipt of Materials	Della Com			11 01 9100
Printed/Typed Name	Signature		_ [Date
	griatero			Month Day Year
19. Discrepancy Indication Space				
	Page			
20. Facility Owner or Operator: Certification of receipt of	<u> </u>	ifact avacat a -	noted in the state	
	74	nest except as	noted in Item 19.	Dr.
Printed/Typed Name		***************************************		Month Day Year

P.O. Box 13087

Austin, Texas 78711-3087



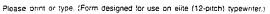
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Please print or type. (Form designed for use on elite (12-pitch) typewriter.)

	to be the state of				1	applicated. Oil	3 140. 2030-0039.	
	WASTE MANIFEST IN	erator's US EPA ID No. 7540024	4728	lanifest ument No.	2. Page of	. !!!!!!!!	nation in the shad required by Fed	
	3. Generator's Name and Mailing Address U.S. ARMY, CHEZATIONS SUPFORT	CMMAND	CHISSON A	<u>e</u>	A. Slate	Manifest Doo	umentNumber 02063	150
	AMSIO-ACE-D ROCK ISLAND	, IL 61299			THE COLUMN TO	Panarataria I	TO I Smith to the second	1 1
	4. Generator's Phone (309) 782 - 033		Gersson					15
	5. Transporter 1 Company Name	10 4 D. O.	EPA ID Number				DEBRE AT	14
	7. Transporter 2 Company Name		EPA ID Number			Transportér's		- 17 / 1 / 1 / 1 / 1 / 1 / 1 / 1 / 1 / 1
					Fi Transi	ortérs Phor	e 法银行 四部	**#.fe4 :
	9. Designated Facility Name and Site Address WASTE CONTROL SPECIALISTS,	LLC 10. US	EPA ID Number		G Stale	Facility's ID	358	
	ITIO W. BROADWAY			, , ,	H, Bacility	s Phone:	· 性器 (基于) 验书	(
	ANDREWS, TX 79714	TXD99	<u> </u>	4.6.4			<i>289-22</i> 8	3∷
	11.A. 11. US DOT Description (including Proper Ship HM Number and Packing Group)	ping Name. Hazard Cla	ss. ID _	12. Contair No.	rers Type	13. Total Quantity	Unit Wt/Vol Was	te-No
	a WASTE, NOD ROPA, NO	D REGULATES)				WI/VOII3	1, 1, 1
ģ	SOIL ADD GRASS (DOT	RADICACTIVE	exempt)	$\varphi.\varphi.$	Cm 2	9330	Pours	319)
Gwzwe	b.	777-010-10-11-0-1	<u> </u>				1 1 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2 2	
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O R	C.						\$3 GG	(diselle
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	5. Special Handling Instructions and Additional Inform	nation WASTE		स्य स्थापित् स्य स्थापित्) • • • • • • • • • • • • • • • • • • •	yddiente i em f		
	VOLUME OF WASTE - 12.2				D-01 =	m 110	ADIUM - 0.3	216-1
	16. GENERATOR'S CERTIFICATION: I hereby declare that		nment are fully a					
	classified, packaged, marked, and labelled/placarded, a national government regulations, including applicable states.		oper condition fo	r transport (y highway	according to a	ppticable internation	nai and
	If I am a large quantity generator. I certify that I have a peconomically practicable and that I have selected the pra	rogram in place to reduce						
	future threat to human health and the environment: OR, if the best waste management method that is available to management method that is available to management method that is available to management method.	il am a small quantity gen		te a good fa				
	Printed/Typed Name	7 OF Signatu	re /	11/1			Month Da	y Yea
1	KIEHAD THATCHER US	Frenu	A1140	ex E	Hick	\bowtie	1/01	8107
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P.O. Box 13087

Austin, Texas 78711-3087





A	UNIFORM HAZARDOUS WASTE MANIFEST 1. Generator'S US EPA ID No. TN 9540024472	Manifest 2. Page 1 Information in the shaded areas is not required by Federal law.
	3. Generator's Name and Mailing Address U.S. ARMY, CPERATIONS Sufficient Command FOR AMSO-ACE-D ROCKISHADD, IL 61299 GEISSIM AHR	A State Mahifest Document Number 02053526
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	7. Transporter 2 Company Name 8. US EPA ID N	4 - 4 - 4 - 4 - 4 - 4 - 4 - 4 - 4 - 4 -
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	9. Designated Facility Name and Site Address WASTE CONTROL SPECIALISTS, LLC 10. US EPA ID N	umber GG/State/Fability's/ID
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-	11.A. 11. US DOT Description (including Proper Shipping Name. Hazard Class. ID Number and Packing Group)	12. Containers 13. 14. Unit No. Type Quantity Wt/Vol. Waste No.
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	J. Additional Descriptions for Materials Listed Above	
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	16. GENERATOR'S CERTIFICATION: I hereby declare that the contents of this consignment are	e fully and accurately described above by proper shipping name and are
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	If I am a large quantity generator. I certify that I have a program in place to reduce the volum economically practicable and that I have selected the practicable method of treatment, storage	
	future threat to human health and the environment; OR, if I am a small quantity generator, I he the best waste management method that is available to me and that I can afford.	ave made a good faith effort to minimize my waste generation and select
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Austin, Texas 78711-3087



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	4. Generator's Phone (309) 782-0338		GassomiAi				99918	
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	7. Transporter 2 Company Name	3. US	EPA ID Number		E State.∏ E Transp	ransporters orters Phon	ID:	
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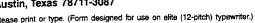
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Form approved. OMB No. 2050-0039.

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	4. Generator's Phone (307) 782 - 0338 5. Transporter 1 Company Name	6. US EPA ID Numb	er U	CiState Trans	14 1 10.	104 (114 ft 11 ft 14 ft
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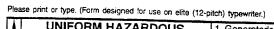
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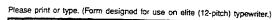
Austin, Texas 78711-3087

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	3. Generator's Name and Mailing Address US Alony, Orbitations Support Command		
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-	4. Generator's Phone (309) 282 – 0338 CLISSON F 5. Transporter 1 Company Name 6. LIS EPA ID Number		
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	7. Transporter 2 Company Name 8. US EPA ID Number		
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